

questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

• *NRC's Agencywide Documents Access and Management System (ADAMS)*: You may obtain publicly-available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. RG 1.236 and the regulatory analysis may be found in ADAMS under Accession Nos. ML20055F490 and ML16124A198, respectively.

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FOR FURTHER INFORMATION CONTACT: Paul Clifford, Office of Nuclear Reactor Regulation, telephone: 301-415-4043, email: Paul.Clifford@nrc.gov and Edward O'Donnell, Office of Nuclear Regulatory Research, telephone: 301-415-3317, email: Edward.ODonnell@nrc.gov. Both are staff of the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

SUPPLEMENTARY INFORMATION:

I. Discussion

The NRC is issuing a new guide in the NRC's "Regulatory Guide" series. This series was developed to describe and make available to the public information regarding methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific issues or postulated events, and information that the staff needs in its review of applications for permits and licenses.

The RG, titled "Pressurized-Water Reactor Control Rod Ejection and Boiling-Water Reactor Control Rod Drop Accidents," describes one acceptable method for demonstrating compliance with appendix A of part 50 of title 10 of the *Code of Federal Regulations* (10 CFR), General Design Criteria (GDC) 28, "Reactivity Limit," with respect to control rod ejection (CRE) accidents for pressurized-water reactors and control rod drop (CRD) accidents for boiling-water reactors. RG 1.236 provides guidance for analyzing these reactivity-initiated accidents. It defines fuel cladding failure thresholds for ductile failure, brittle failure, and pellet-clad mechanical interaction. It also describes analytical limits and guidance for demonstrating compliance with regulations governing reactivity limits.

The guide incorporates new empirical data from in-pile, prompt power pulse test programs and analyses from several international publications that examine fuel rod performance under reactivity-initiated accident conditions to provide guidance on acceptable analytical methods, assumptions, and limits for evaluating a CRE and CRD accidents. The guide supersedes the guidance for control rod ejection accidents in RG 1.77, "Assumptions Used for Evaluation of a Control Rod Ejection Accident for Pressurized Water Reactors," which is being withdrawn in conjunction with the issuance of RG 1.236.

II. Additional Information

The NRC issued RG 1.236 as Draft Guide (DG)-1327 for public comment on November 21, 2016 (81 FR 83288), with a 60 day comment period that expired on February 21, 2017. A public meeting was held at NRC Headquarters on January 25, 2017, while the guide was issued for public comment. During the meeting NRC made a commitment to hold a second public meeting to discuss the staff's proposed resolution of key comments prior to finalization of the guide. Following the public meeting the NRC extended the comment period to April 21, 2017 (82 FR 8998), to allow more time for comment. A second public meeting was held at NRC Headquarters on June 5, 2018, to discuss resolution of the public comments. To facilitate discussion at the meeting, drafts of the guide (ADAMS Accession No. ML18138A459) and a table showing resolution of the public comments (ADAMS Accession No. ML18138A458) were made publicly available.

As a result of the written public comments and discussion at the two public meetings, the NRC made several changes to the draft guide and it was released a second time for public comment on July 30, 2019 (84 FR 36961). That comment period ended November 18, 2019. The guide was further revised to address the second round of public comments. The response to this second round of public comments is located in ADAMS under Accession No. ML20055F489. Among the changes resulting from the two public meetings and the two rounds of public comments were: (1) The pellet-clad mechanical interaction cladding failure thresholds were revised; (2) the appendix that provided acceptable steady-state and transient fission product releases was removed; and (3) the implementation section was revised to be consistent with recent Commission direction on backfitting and forward fitting as found in Management Directive 8.4 "Management of

Backfitting, Forward Fitting, Issue Finality, and Information Requests."

III. Congressional Review Act

This RG is a rule as defined in the Congressional Review Act (5 U.S.C. 801-808). However, the Office of Management and Budget has not found it to be a major rule as defined in the Congressional Review Act.

IV. Backfitting, Forward Fitting, and Issue Finality

RG 1.236 describes one acceptable method for demonstrating compliance with 10 CFR part 50, appendix A, GDC 28 with respect to CRE accidents for pressurized-water reactors and CRD accidents for boiling-water reactors. In general, issuance of this RG does not constitute backfitting as defined in 10 CFR 50.109, "Backfitting," and as described in NRC Management Directive 8.4; does not constitute forward fitting as that term is defined and described in Management Directive 8.4; and does not affect the issue finality of any approval issued under 10 CFR part 52, "Licenses, Certificates, and Approvals for Nuclear Power Plants." As explained in this regulatory guide, applicants and licensees are not required to comply with the positions set forth in this RG. Licensees using RG 1.77 may continue using that RG. In future requests or applications for NRC licensing actions related to the guidance in RG 1.77, those licensees should use RG 1.236, which contains the guidance in RG 1.77. Further information on the staff's use of the RG is contained in the RG under Section D., "Implementation."

Dated: June 16, 2020.

For the Nuclear Regulatory Commission.

Meraj Rahimi,

Chief, Regulatory Guidance and Generic Issues Branch, Division of Engineering, Office of Nuclear Regulatory Research.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-331; NRC-2020-0148]

NextEra Energy Duane Arnold, LLC; Duane Arnold Energy Center; Post-Shutdown Decommissioning Activities Report

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of receipt; availability; public meeting and request for comment.

SUMMARY: On April 2, 2020, the U.S. Nuclear Regulatory Commission (NRC)

received the post-shutdown decommissioning activities report (PSDAR) for the Duane Arnold Energy Center (DAEC). The PSDAR, which includes the site-specific decommissioning cost estimate (DCE), provides an overview of NextEra Energy Duane Arnold, LLC's (NEDA or the licensee) planned decommissioning activities, schedule, projected costs, and environmental impacts for DAEC. The NRC will hold a public meeting to discuss the PSDAR's content and receive comments.

DATES: Submit comments by October 19, 2020. Comments received after this date will be considered if it is practical to do so, but the NRC is able to ensure consideration only for comments received on or before this date.

ADDRESSES: You may submit comments by any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0148. Address questions about NRC docket IDs to Jennifer Borges; telephone: 301-287-9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *Mail comments to:* Office of Administration, Mail Stop: TWFN-7-A60M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, ATTN: Program Management, Announcements and Editing Staff.

For additional direction on obtaining information and submitting comments, see "Obtaining Information and Submitting Comments" in the **SUPPLEMENTARY INFORMATION** section of this document.

FOR FURTHER INFORMATION CONTACT: Scott P. Wall, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001, telephone: 301-415-2855; email: Scott.Wall@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Obtaining Information and Submitting Comments

A. Obtaining Information

Please refer to Docket ID NRC-2020-0148 when contacting the NRC about the availability of information for this action. You may obtain publicly-available information related to this action by any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0148.
- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-

available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

B. Submitting Comments

Please include Docket ID NRC-2020-0148 in your comment submission.

The NRC cautions you not to include identifying or contact information that you do not want to be publicly disclosed in your comment submission. The NRC will post all comment submissions at <https://www.regulations.gov> as well as enter the comment submissions into ADAMS. The NRC does not routinely edit comment submissions to remove identifying or contact information.

If you are requesting or aggregating comments from other persons for submission to the NRC, then you should inform those persons not to include identifying or contact information that they do not want to be publicly disclosed in their comment submission. Your request should state that the NRC does not routinely edit comment submissions to remove such information before making the comment submissions available to the public or entering the comment submissions into ADAMS.

II. Discussion

NEDA is the holder of Renewed Facility Operating License No. DPR-49 for DAEC. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the NRC now or hereafter in effect. The facility consists of one boiling-water reactor located adjacent to the Cedar River approximately 2.5 miles northeast of the town of Palo in Linn County, Iowa. By letter dated January 18, 2019 (ADAMS Accession No. ML19023A196), the licensee submitted its notification of permanent cessation of power operations for DAEC. In this letter, NEDA notified the NRC of its intent to permanently cease power operations at DAEC in the fourth quarter of 2020. By letter dated March 2, 2020 (ADAMS Accession No. ML20062E489), the licensee submitted a revised notification of permanent cessation of power operations for DAEC. In this letter, NEDA notified the NRC of its intent to

permanently cease power operations at DAEC on October 30, 2020.

On April 2, 2020, NEDA submitted the PSDAR, including the site-specific DCE for DAEC, in accordance with § 50.82(a)(4)(i) of title 10 of the *Code of Federal Regulations* (ADAMS Accession No. ML20094F603). The PSDAR includes a description of the planned decommissioning activities, a proposed schedule for their accomplishment, the site-specific DCE, and a discussion that provides the basis for the NRC to conclude that the environmental impacts associated with the site-specific decommissioning activities will be bounded by appropriate, previously issued generic and plant-specific environmental impact statements.

III. Request for Comment and Public Meeting

Due to the COVID-19 Public Health Emergency (PHE), the NRC has delayed holding a public meeting to obtain comments from the public on the DACE PSDAR, including the DCE. However, with recent announcements for relaxing PHE restrictions by Iowa Governor Reynolds, the NRC is tentatively planning to hold the PSDAR meeting and receive comments on Tuesday, July 28, 2020, from 6 p.m. until 9 p.m., at the Palo Community Center, 2800 Hollenbeck Rd, Palo, Iowa 52324. If PHE restrictions do not allow the meeting to be held on July 28, 2020, the NRC will notify the public by revising the date on the NRC Public Meeting Schedule on the NRC's public website at <https://www.nrc.gov/pmns/mtg>. The NRC requests that comments provided outside the July 28, 2020 meeting be submitted as noted in Section I, "Obtaining Information and Submitting Comments," of this document in writing by October 19, 2020.

Dated: June 15, 2020.

For the Nuclear Regulatory Commission.

Nancy L. Salgado,

Chief, Plant Licensing Branch III, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

[NRC-2019-0248]

Information Collection: Requests to Agreement States for Information

AGENCY: Nuclear Regulatory Commission.

ACTION: Renewal of existing information collection; request for comment.