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NextEra Energy Seabrook, LLC, Docket No. 50-443, Seabrook Station, Unit No. 1, Rockingham County, New Hampshire

Date of amendment request: April 4, 2017, as supplemented by letter dated April 8, 2017.

Description of amendment request: The amendment is a one-time change to the licensing basis for the service water cooling tower, which provides the

standby seismically qualified ultimate heat sink for Seabrook Station, Unit No. 1, to be removed from service for maintenance on the cooling tower basin with the reactor plant in operational Modes 5 or 6, cold shutdown or refueling, respectively, during the April 2017 refueling outage. During the maintenance period, the normal heat sink provided by the non-seismic tunnel access to the Atlantic Ocean would remain in service.

Date of issuance: April 13, 2017.

Effective date: This license amendment is effective as of its date of issuance and shall be implemented immediately for the period that Seabrook Station, Unit No. 1, is in Modes 5 and 6 during the April 2017 refueling outage.

Amendment No.: 155. A publicly-available version is in ADAMS under Accession No. ML17102A889; documents related to this amendment are listed in the Safety Evaluation enclosed with the amendment.

Facility Operating License No. NPF-86: Amendment revised the Facility Operating License licensing basis.

Public comments requested as to proposed no significant hazards consideration (NSHC): Yes. The *Portsmouth Herald* and *The Boston Globe* on April 10, 2017, and April 11, 2017. The notice provided an opportunity to submit comments on the Commission's proposed NSHC determination. A public comment was received and addressed in the Safety Evaluation.

The Commission's related evaluation of the amendment, finding of exigent circumstances, state consultation, and final NSHC determination are contained in a Safety Evaluation dated April 13, 2017.

Attorney for licensee: William Blair, Managing Attorney—Nuclear Florida Power & Light Company, P.O. Box 14000, Juno Beach, FL 33408-0420.

NRC Branch Chief: James G. Danna.

Dated at Rockville, Maryland, this 28th day of April 2017.

For the Nuclear Regulatory Commission.

Kathryn M. Brock,

Deputy Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2017-09345 Filed 5-8-17; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[NRC-2017-0104]

Biweekly Notice: Applications and Amendments to Facility Operating Licenses and Combined Licenses Involving No Significant Hazards Considerations; Correction

AGENCY: Nuclear Regulatory Commission.

ACTION: Biweekly notice; correction.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is correcting a notice that was published in the **Federal Register** (FR) on April 25, 2017, regarding notice of issuance of amendments to facility operating licenses and combined licenses. This action is necessary to correct an administrative error.

DATES: The correction is effective May 9, 2017.

ADDRESSES: Please refer to Docket ID NRC-2017-0104 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2017-0104. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov.

- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT: V. Sreenivas, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-415-2597, email: V.Sreenivas@nrc.gov.

SUPPLEMENTARY INFORMATION: In the FR on April 25, 2017 (82 FR 19095), FR Doc. 2017–08115, on page 19108, under Exelon Generation Company, LLC, Docket No. 50–353, Limerick Generating Station, Unit 2, Montgomery County, Pennsylvania, in the third column, paragraph 4, line 11, “Amendment No.: 186” is corrected to read “Amendment No.: 187.”

Dated at Rockville, Maryland, this 27th day of April 2017.

For the Nuclear Regulatory Commission.

V. Sreenivas,

*Project Manager, Plant Licensing Branch I,
Division of Operating Reactor Licensing,
Office of Nuclear Reactor Regulation.*

[FR Doc. 2017–09367 Filed 5–8–17; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–498 and 50–499; NRC–2016–0092]

STP Nuclear Operating Company, South Texas Project, Units 1 and 2

AGENCY: Nuclear Regulatory Commission.

ACTION: Environmental assessment and finding of no significant impact; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing an environmental assessment (EA) prepared under the National Environmental Policy Act of 1969 (NEPA) and NRC’s regulations. This EA summarizes the results of the NRC staff’s environmental review, which evaluates the potential environmental impacts of granting exemptions from NRC regulations in response to a request from STP Nuclear Operating Company (STPNOC, the licensee) for Facility Operating License Nos. NPF–76 and NPF–80, for South Texas Project (STP), Units 1 and 2, respectively, located in Matagorda County, Texas. The regulatory exemptions, if granted, allow STPNOC to change the licensing basis loss-of-coolant accident analysis identified in the Updated Final Safety Analysis Report to use a risk-informed approach to address safety issues discussed in Generic Safety Issue (GSI)–191 and to close Generic Letter (GL) 2004–02.

DATES: May 9, 2017.

ADDRESSES: Please refer to Docket ID NRC–2016–0092 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available

information related to this document using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC–2016–0092. Address questions about NRC dockets to Carol Gallagher; telephone: 301–415–3463; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

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- *NRC’s PDR:* You may examine and purchase copies of public documents at the NRC’s PDR, Room 01–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT: Lisa Regner, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415–1906, email: Lisa.Regner@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction and Background

The NRC is considering a request to grant certain regulatory exemptions for Facility Operating License Nos. NPF–76 and NPF–80, issued to STPNOC for operation of STP, Units 1 and 2, located in Matagorda County, Texas, in accordance with section 50.12, “Specific exemptions,” of title 10 of the *Code of Federal Regulations* (10 CFR), “Application for amendment of license, construction permit, or early site permit.” The regulatory exemptions would allow STPNOC to resolve concerns associated with GSI–191, “Assessment of Debris Accumulation on PWR [Pressurized-Water Reactor] Sump Performance,” and the associated GL 2004–02, “Potential Impact of Debris Blockage on Emergency Recirculation during Design Basis Accidents at Pressurized-Water Reactors,” issued on September 13, 2004.

Pursuant to 10 CFR 51.21, “Criteria for and identification of licensing and

regulatory actions requiring environmental assessments,” the NRC has prepared an EA summarizing the findings of its NEPA review of the proposed action. The NRC concluded that the proposed action will have no significant environmental impact.

The NRC published a draft EA on the proposed action for public comment in the **Federal Register** on May 4, 2016 (81 FR 26838) (ADAMS Accession No. ML16032A387). No comments were received.

Background

The NRC established GSI–191 to determine whether the transport and accumulation of debris from a loss-of-coolant accident in the PWR containment structure would impede the operation of the emergency core cooling system (ECCS) or containment spray system (CSS). A loss-of-coolant accident within the containment structure is assumed to be caused by a break in the primary coolant loop piping. Water discharged from the pipe break would collect on the containment structure floor and within the containment emergency sump. During this type of accident, the ECCS and CSS would initially draw cooling water from the refueling water storage tank. However, realigning the ECCS pumps to the containment structure emergency sump would provide long-term cooling of the reactor core. Therefore, successful long-term cooling depends on the ability of the containment structure emergency sump to provide adequate flow to the residual heat removal recirculation pumps for extended periods of time.

One of the concerns addressed by the implementation of GSI–191 is that debris, such as insulation installed on piping and components, within the containment structure could be dislodged by a jet of water and steam from a loss-of-coolant accident. Water, along with debris, would accumulate at the bottom of the containment structure and flow towards the emergency sump pumps. Insulation and other fibrous material could block the emergency sump screens and suction strainers, which in turn could prevent the ability of the containment emergency sump to provide adequate flow to the residual heat removal recirculation pumps (for more information, see NUREG–0897, “Containment Emergency Sump Performance”).

The NRC issued GL 2004–02 to address this safety concern by requesting PWR licensees, pursuant to 10 CFR 50.54(f), to use an NRC-approved methodology to perform a “mechanistic evaluation of the potential for the adverse effects of post-accident