Day	Event/Activity
25	If NRC staff finds no ''need'' or no likelihood of standing, the deadline for petitioner/requester to file a motion seeking a ruling to reverse the NRC staff's denial of access; NRC staff files copy of access determination with the presiding officer (or Chief Administrative Judge or other designated officer, as appropriate). If NRC staff finds ''need'' for SUNSI, the deadline for any party to the proceeding whose interest independent of the proceeding would be harmed by the release of the information to file a motion seeking a ruling to reverse the NRC staff's grant of access.
30	Deadline for NRC staff reply to motions to reverse NRC staff determination(s).
40	(Receipt + 30) If NRC staff finds standing and need for SUNSI, deadline for NRC staff to complete information processing and file motion for Protective Order and draft Non-Disclosure Affidavit. Deadline for applicant/licensee to file Non-Disclosure Agreement for SUNSI.
A	If access granted: Issuance of presiding officer or other designated officer decision on motion for protective order for access to sensitive information (including schedule for providing access and submission of contentions) or decision reversing a final adverse determination by the NRC staff.
A + 3	Deadline for filing executed Non-Disclosure Affidavits. Access provided to SUNSI consistent with decision issuing the protective order.
A + 28	Deadline for submission of contentions whose development depends upon access to SUNSI. However, if more than 25 days remain between the petitioner's receipt of (or access to) the information and the deadline for filing all other contentions (as established in the notice of hearing or opportunity for hearing), the petitioner may file its SUNSI contentions by that later deadline.
A + 53	(Contention receipt + 25) Answers to contentions whose development depends upon access to SUNSI.
A + 60	(Answer receipt + 7) Petitioner/Intervenor reply to answers.
>A + 60	Decision on contention admission.

[FR Doc. 2016–12484 Filed 6–6–16; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC–2016–0001]

Sunshine Act Meeting Notice

DATES: June, 6, 13, 20, 27, July 4, 11, 2016.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and Closed.

Week of June 6, 2016

There are no meetings scheduled for the week of June 6, 2016.

Week of June 13, 2016—Tentative

There are no meetings scheduled for the week of June 13, 2016.

Week of June 20, 2016—Tentative

Monday, June 20, 2016

9:00 a.m. Meeting with Department of Energy Office of Nuclear Energy (Public Meeting); (Contact: Albert Wong: 301–415–3081).

This meeting will be webcast live at the Web address—*http://www.nrc.gov/.*

Thursday, June 23, 2016

9:00 a.m. Discussion of Security Issues (Closed Ex. 3).

Week of June 27, 2016—Tentative

Tuesday, June 28, 2016

9:30 a.m. Briefing on Human Capital and Equal Opportunity Employment (Public Meeting); (Contact: Kristin Davis: 301–287–0707).

This meeting will be webcast live at the Web address—*http://www.nrc.gov/.*

Week of July 4, 2016—Tentative

Thursday, July 7, 2016

9:30 a.m. Strategic Programmatic Overview of the Reactors Operating Business Line (Public Meeting); (Contact: Trent Wertz: 301–415–1568).

Week of July 11, 2016—Tentative

There are no meetings scheduled for the week of July 11, 2016.

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The schedule for Commission meetings is subject to change on short notice. For more information or to verify the status of meetings, contact Denise McGovern at 301–415–0681 or via email at *Denise.McGovern@nrc.gov.*

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The NRC Commission Meeting Schedule can be found on the Internet at: *http://www.nrc.gov/public-involve/public-meetings/schedule.html.*

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The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (*e.g.* braille, large print), please notify Kimberly Meyer, NRC Disability Program Manager, at 301–287–0739, by videophone at 240–428–3217, or by email at *Kimberly.Meyer-Chambers@nrc.gov.* Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

* * * * *

Members of the public may request to receive this information electronically. If you would like to be added to the distribution, please contact the Nuclear Regulatory Commission, Office of the Secretary, Washington, DC 20555 (301–415–1969), or email *Brenda.Akstulewicz@nrc.gov* or *Patricia.Jimenez@nrc.gov.*

Dated: June 2, 2016.

Denise L. McGovern,

Policy Coordinator, Office of the Secretary.

[FR Doc. 2016–13563 Filed 6–3–16; 4:15 pm]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC–2016–0097]

Consequential SGTR Analysis for Westinghouse and Combustion Engineering Plants With Thermally-Treated Alloy 600 and 690 Steam Generator Tubes

AGENCY: Nuclear Regulatory Commission.

ACTION: Draft NUREG; request for comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing for public comment a draft NUREG, NUREG–2195, ''Consequential SGTR Analysis for Westinghouse and Combustion Engineering Plants with Thermally Treated Alloy 600 and 690 Steam Generator Tubes.'' This report summarizes severe accident-induced consequential steam generator tube rupture (C–SGTR) analyses recently

performed by the NRC's Office of Nuclear Regulatory Research. The analyses described in this report include risk assessment, thermal-hydraulic analyses, and materials behavior analyses.

DATES: Submit comments by August 8, 2016. Comments received after this date will be considered if it is practical to do so, but the Commission is able to ensure consideration only for comments received before this date.

ADDRESSES: You may submit comments by any of the following methods (unless this document describes a different method for submitting comments on a specific subject):

- *Federal Rulemaking Web site:* Go to *http://www.regulations.gov* and search for Docket ID NRC–2016–0097. Address questions about NRC dockets to Carol Gallagher; telephone: 301–415–3463; email: *Carol.Gallagher@nrc.gov.* For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.
- *Mail comments to:* Cindy Bladey, Office of Administration, Mail Stop: OWFN–12–H08, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

For additional direction on accessing information and submitting comments, see ''Obtaining Information and Submitting Comments'' in the **SUPPLEMENTARY INFORMATION** section of this document.

FOR FURTHER INFORMATION CONTACT: Selim Sancaktar, Office of Nuclear Regulatory Research; U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415–2391; email: *Selim.Sancaktar@nrc.gov.*

SUPPLEMENTARY INFORMATION:

I. Obtaining Information and Submitting Comments

A. Obtaining Information

Please refer to Docket ID NRC–2016–0097 when contacting the NRC about the availability of information for this action. You may obtain publicly-available information related to this action by any of the following methods:

- *Federal Rulemaking Web site:* Go to *http://www.regulations.gov* and search for Docket ID NRC–2016–0097.
- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at *http://www.nrc.gov/reading-rm/adams.html.* To begin the search, select ''*ADAMS Public Documents''* and then select ''*Begin Web-based ADAMS Search.''* For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to *pdr.resource@nrc.gov.* Draft NUREG–2195 can be found in ADAMS under at Accession No. ML16134A029.
- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

B. Submitting Comments

Please include Docket ID NRC–2016–0097 in your comment submission.

The NRC cautions you not to include identifying or contact information that you do not want to be publicly disclosed in your comment submission. The NRC posts all comment submissions at *http://www.regulations.gov* as well as entering the comment submissions into ADAMS. The NRC does not routinely edit comment submissions to remove identifying or contact information.

If you are requesting or aggregating comments from other persons for submission to the NRC, then you should inform those persons not to include identifying or contact information that they do not want to be publicly disclosed in their comment submission. Your request should state that the NRC does not routinely edit comment submissions to remove such information before making the comment submissions available to the public or entering the comment submissions into ADAMS.

II. Discussion

This report summarizes severe accident-induced consequential steam generator tube rupture (C–SGTR) analyses recently performed by the NRC's Office of Nuclear Regulatory Research. The C–SGTRs are potentially risk-significant events because thermally-induced steam generator (SG) tube failures caused by hot gases from a damaged reactor core can result in a containment bypass event and a large release of fission products to the environment. The main accident scenarios of interest are those that lead to core damage with high reactor pressure, dry SG, and low SG pressure (high-dry-low) conditions. A typical example of such an accident scenario is a station blackout with loss of auxiliary feedwater. The analyses described in this report include risk assessment, thermal-hydraulic analyses, and materials behavior analyses. This work builds on, and updates, previous NRC work.

The current analyses evaluate replacement SGs with thermally-treated Alloy 600 and Alloy 690 heat exchange tubes and use the latest tube flaw data available in the 2010 time frame. A main focus of this work was to compare C–SGTR results for the different SG geometries associated with Westinghouse and Combustion Engineering plant designs. It has been previously understood that the geometry of the SG reactor coolant inlet plenum region and the hot-leg (HL) influences the temperature of the gases reaching the steam generator tubes during closed-loop-seal natural circulation conditions. Hotter gases reaching the SG tube reduce the time before tube failure, which increases the likelihood of containment bypass. However, if a thermally-induced failure sufficient to depressurize the reactor coolant system (RCS) develops in another location, fission product release through failed SG tubes may be prevented or minimized. Therefore, the possibility of an earlier failure of other RCS components (such as the reactor coolant HL) is also considered. Pressure-induced steam generator tube rupture (SGTR) scenarios, which also may lead to tube failure and subsequent containment bypass, were also studied, but are deemed to be of lesser potential impact on overall plant risk.

The methods developed were intended to address the contribution of thermally-induced SGTR during severe accidents and pressure-induced SGTR during a number of design-basis accidents. The methods and the pilot applications were developed in a manner that can establish the framework to perform a more comprehensive Probabilistic Risk Assessment that can address the C–SGTR at a level of detail suitable for other NRC needs.

Dated at Rockville, Maryland, this 26th day of May 2016.

For the Nuclear Regulatory Commission.

Kevin Coyne,

Branch Chief, Probabilistic Risk Assessment Branch, Division of Risk Analysis, Office of Nuclear Regulatory Research.

[FR Doc. 2016–13387 Filed 6–6–16; 8:45 am]

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