

understood, and the impact of collection requirements on respondents can be properly assessed.

A copy of the proposed information collection request can be obtained by contacting the individual listed below in the **ADDRESSES** section of this notice.

**DATES:** Comments must be submitted to the office listed in the **FOR FURTHER INFORMATION CONTACT** section below on or before August 13, 2012.

OMB is particularly interested in comments that help the agency to:

- Evaluate whether the proposed collection of information is necessary for the proper performance of the functions of the agency, including whether the information will have practical utility;
- Evaluate the accuracy of the agency's estimate of the burden of the proposed collection of information, including the validity of the methodology and assumptions used;
- Enhance the quality, utility, and clarity of the information to be collected; and
- Minimize the burden of the collection of information on those who are to respond, including through the use of appropriate automated, electronic, mechanical, or other technological collection techniques or other forms of information technology, e.g., permitting electronic submission of responses.

**ADDRESSES:** Kim A. Miller, Management Analyst, Institute of Museum and Library Services, 1800 M Street NW., 9th Floor, Washington, DC 20036. Telephone: 202-653-4762; Fax: 202-653-4600; or email: [kmiller@imls.gov](mailto:kmiller@imls.gov); or by teletype (TTY/TDD) for persons with hearing difficulty at 202-653-4614.

**SUPPLEMENTARY INFORMATION:** The Institute of Museum and Library Services is the primary source of federal support for the Nation's 123,000 libraries and 17,500 museums. The mission of IMLS is to inspire libraries and museums to advance innovation, lifelong learning, and cultural and civic engagement. We provide leadership through research, policy development, and grant making. IMLS provides a variety of grant programs to assist the Nation's museums and libraries in improving their operations and enhancing their services to the public. (20 U.S.C. 9101 *et seq.*)

*Current Actions:* This notice proposes general clearance of the agency's guideline application and report forms. The 60-day Notice for the "Notice of Continuance for General Clearance for Guidelines, Applications, and Reporting Forms" was published in the **Federal Register** on May 10, 2012 (FR vol. 77,

No. 91, pgs. 27486). No comments were received.

*Agency:* Institute of Museum and Library Services.

*Title:* IMLS Guidelines, Applications and Reporting Forms.

*OMB Number:* 3137-0029, 3137-0071.

*Agency Number:* 3137.

*Frequency:* Annually, Semi-annually.

*Affected Public:* State Library Administrative Agencies, museums, libraries, institutions of higher education, library and museum professional associations, and museum and library professionals, Indian tribes (including Alaska native villages, regional corporations, or village corporations), and organizations that primarily serve and represent Native Hawaiians.

*Number of Respondents:* 7,961.

*Estimated Time per Respondent:* .08-90 hours.

*Total Burden Hours:* 70,092.

*Total Annualized Capital/Startup Costs:* 0.

*Total Annual Costs:* \$1,921,209.

**FOR FURTHER INFORMATION CONTACT:**

Comments should be sent to Office of Information and Regulatory Affairs, Attn.: OMB Desk Officer for Education, Office of Management and Budget, Room 10235, Washington, DC 20503, (202) 395-7316.

Dated: July 10, 2012.

**Kim A. Miller,**

*Management Analyst, Office of Policy, Planning, Research, and Communication.*

[FR Doc. 2012-17169 Filed 7-12-12; 8:45 am]

**BILLING CODE 7036-01-P**

**NUCLEAR REGULATORY COMMISSION**

[Docket No. 50-425; NRC-2012-0169]

**Southern Nuclear Operating Company, Inc.; Notice of Withdrawal of Application for Amendment to Facility Operating License**

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Southern Nuclear Operating Company, Inc. (the licensee) to withdraw its December 19, 2011, application for proposed amendment to Facility Operating License No. NPF-81 for the Vogtle Electric Generating Plant, Unit 2, located in Burke County, Georgia.

The proposed amendment would have revised the Technical Specifications related to the Engineered Safety Features Room Cooler and Safety-Related Chiller System, Allowed Completion Time for Condition A.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on February 7, 2012 (77 FR 6149). However, by letter dated June 19, 2012, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated December 19, 2011, and the licensee's letter dated June 19, 2012, which withdrew the application for license amendment. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland 20852. Publicly available documents created or received at the NRC are accessible electronically through the Agencywide Documents Access and Management System (ADAMS) in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, or 301-415-4737 or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

Dated at Rockville, Maryland, this 5th day of July 2012.

For the Nuclear Regulatory Commission.

**Patrick G. Boyle,**

*Project Manager, Plant Licensing Branch II-1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2012-17121 Filed 7-12-12; 8:45 am]

**BILLING CODE 7590-01-P**

**NUCLEAR REGULATORY COMMISSION**

[Docket Nos. 50-247 and 50-286; NRC-2012-0168]

**Entergy Nuclear Indian Point Unit 2, LLC, Entergy Nuclear Indian Point Unit 3, LLC, Entergy Nuclear Operations, Inc., Indian Point Nuclear Generating Units 2 and 3; Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment and changes to the Technical Specifications (TSs) for Facility Operating License Nos. DPR-26 and DPR-64, issued to Entergy Nuclear Operations, Inc. (Entergy or the licensee) for operation of the Indian Point Nuclear Generating Units 2 and 3 (IP2 and IP3) located in Westchester County, New York, in accordance with Title 10 of the *Code of Federal*

*Regulations* (10 CFR) 50.90. The proposed changes request NRC approval for the transfer of spent fuel from the IP3 spent fuel pool (SFP) to the IP2 SFP using a newly-designed shielded transfer canister (STC), for further transfer to the on-site Independent Spent Fuel Storage Installation (ISFSI). Therefore, as required by 10 CFR 51.21, the NRC staff performed an environmental assessment (EA). The NRC staff did not identify any significant environmental impacts associated with the proposed action based on its evaluation of the information provided in the licensee's application and other available information. Therefore, the NRC staff is issuing a finding of no significant impact (FONSI) for the proposed action.

#### **Environmental Assessment**

##### **Plant Site and Environs**

IP2 and IP3 are located on approximately 239 acres (97 hectares) of land in the Village of Buchanan in upper Westchester County, New York. The facility is on the eastern bank of the Hudson River. Both IP2 and IP3 use Westinghouse pressurized-water reactors and nuclear steam supply systems. For each unit, cooling is provided by a once-through cooling water intake that supplies cooling water from the Hudson River. Indian Point Nuclear Generating Unit No. 1 (IP1), now permanently shut down, shares the site with IP2 and IP3. IP1 was shut down in 1974, and is in a safe storage condition awaiting final decommissioning.

##### **Identification of the Proposed Action**

The proposed changes request NRC approval for the transfer of spent fuel from the IP3 SFP to the IP2 SFP using a newly-designed STC, for further transfer to the on-site ISFSI, which uses the Holtec HI-STORM 100 dry cask storage system that has been previously certified for dry spent fuel storage under 10 CFR part 72. Entergy has no plans to make extensive physical modifications to existing plant buildings or property for the proposed action. The proposed action is detailed in the licensee's application dated July 8, 2009, Agencywide Documents Access and Management System (ADAMS) Accession No. ML091940176, as supplemented by letters dated September 28, 2009; ADAMS Accession No. ML092950437; October 26, 2009, ADAMS Accession No. ML093020080; October 5, 2010, ADAMS Accession No. ML102910511; October 28, 2010, ADAMS Accession Nos. ML103080112 and ML103080113; July 28, 2011,

ADAMS Accession No. ML11220A079; August 23, 2011, ADAMS Accession Nos. ML11243A174, ML11243A175; and ML11243A220; October 28, 2011, ADAMS Accession No. ML11327A045 and ML11327A046; December 15, 2011, ADAMS Accession No. ML12013A259; January 11, 2012, ADAMS Accession No. ML120400604; March 2, 2012, ADAMS Accession No. ML12074A027, April 23, 2012, ADAMS Accession No. ML12129A457, and May 7, 2012, ADAMS Accession No. ML121370318. The licensee's application and supplemental submissions are accessible electronically from the NRC's Web site, [www.nrc.gov](http://www.nrc.gov).

##### **The Need for the Proposed Action**

Entergy requested the proposed action because transferring the IP3 spent fuel from the IP3 SFP directly into dry storage casks is not possible due to the limitations of the 40-ton cask handling crane in the IP3 fuel storage building (FSB) where the SFP is located. A cask handling crane capacity of at least 100 tons is required to lift and handle the loaded HI-TRAC transfer cask licensed as part of the HI-STORM 100 System. Entergy had previously added a single-failure-proof gantry crane with this capacity to the IP2 FSB, by excavating to bedrock and supporting the crane foundation on bedrock. An upgrade to the IP3 cask handling crane capacity to 100 tons or more was evaluated and found to be not feasible and as such results in the need for inter-unit fuel transfer. The IP3 SFP is approaching the limit of its storage capacity. Spent fuel must be removed from the IP3 SFP to restore and maintain the ability to unload the entire IP3 reactor core into the IP3 SFP for the remainder of its service life in order to perform maintenance on the reactor vessel and associated systems.

##### **Environmental Impacts of the Proposed Action**

###### **Non-Radiological Impacts**

###### **Land Use and Aesthetic Impacts**

There are no potential land use and aesthetic impacts from the proposed action. No new construction of buildings is proposed. The work activities would occur within existing structures. Existing parking lots, road access, equipment lay-down areas, offices, workshops, warehouses, and restrooms would be used during implementation of the proposed action. Land use conditions would not change at the Indian Point site. Therefore, there would be no significant impact from the proposed action.

###### **Air Quality Impacts**

Some minor and short duration air quality impacts would occur during implementation of the fuel transfer at the site. The main source of air emissions would come from the vehicles driven by plant workers and contractors. However, air emissions would be less than is experienced during the routine refueling outages once each year. Therefore, there would be no significant impact on air quality in the region during and following implementation of the proposed action.

###### **Surface Water Impacts**

There are no potential surface water impacts from the proposed action. No new use of surface water or effluent discharges into surface water will be made as part of the proposed action. Therefore, there would be no significant impact to surface water resources during implementation of the proposed action.

###### **Groundwater Impacts**

There are no potential groundwater impacts from the proposed action. No new use of groundwater or effluent discharges into groundwater will be made as part of the proposed action. Therefore, there would be no significant impact to groundwater resources during implementation of the proposed action.

###### **Aquatic Resources Impacts**

There are no potential impacts to aquatic resources from the proposed action. No new effluent discharges into the aquatic environment will be made as part of the proposed action. Therefore, there would be no significant impact to aquatic resources during implementation of the proposed action.

###### **Terrestrial Resources Impacts**

There are no potential impacts to terrestrial resources from the proposed action. No new land areas will be disturbed and no new effluent discharges will be made as part of the proposed action. Therefore, there would be no significant impact to terrestrial resources during implementation of the proposed action.

###### **Threatened and Endangered Species Impacts**

There are no potential impacts to threatened and endangered species from the proposed action. No new withdrawals from the Hudson River or any new effluent discharges into the aquatic environment will be made as part of the proposed action. Therefore, there would be no significant impact to threatened and endangered species during implementation of the proposed action.

### Historic and Archaeological Resources Impacts

There are no potential impacts to historic and archaeological resources from the proposed action because no new construction on the site or vicinity of the site is proposed. The work activities would occur within existing structures. Existing parking lots, road access, equipment lay-down areas, offices, workshops, warehouses, and restrooms would be used during implementation of the proposed action. Therefore, there would be no significant impact to historic and archaeological resources from the proposed action.

### Socioeconomic Impacts

Potential socioeconomic impacts from the proposed action include a temporary increase in the size of the workforce at the Indian Point site. The expected increase is much smaller than the additional workforce experienced during a refueling outage. Therefore, due to the small and temporary increase in the number of workers needed to support the proposed action, there are no significant socioeconomic impacts associated with the proposed action.

### Environmental Justice Impacts

The environmental justice impact analysis evaluates the potential for disproportionately high and adverse human health and environmental effects on minority and low-income populations that could result from activities associated with the proposed action at the Indian Point site. Such effects may include human health, biological, cultural, economic, or social impacts. Minority and low-income populations are subsets of the general population residing in the vicinity of the Indian Point site, and all are exposed to the same health and environmental effects generated from activities at the Indian Point site. Based on this information and the analysis of human health and environmental impacts presented in this environmental assessment, the proposed action would not have disproportionately high and adverse human health and environmental effects on minority and low-income populations residing in the vicinity of the Indian Point site.

### Radiological Impacts

#### Radioactive Gaseous and Liquid Effluents and Solid Waste

Indian Point uses waste treatment systems to collect, process, recycle, and dispose of gaseous, liquid, and solid wastes that contain radioactive material in a safe and controlled manner within NRC and Environmental Protection

Agency radiation safety standards. The proposed action will not significantly change the types or amounts of radioactive gaseous and liquid waste. At the site, the volume of solid radioactive waste is expected to show a small increase because of the use of protective clothing for the workers, the disposal of used seals from the STC and HI-TRAC lids, and decontamination work performed on equipment and work areas. However, the additional volume would not have a significant effect on the plant's ability to handle and process the waste. Based on the above, there are no significant radioactive waste impacts associated with the proposed action.

### Occupational Radiation Dose

To protect plant workers, the licensee's radiation protection program monitors radiation levels throughout the plant to establish appropriate work controls, training, temporary shielding, and protective equipment requirements so that worker doses will remain within the dose limits of 10 CFR part 20. Entergy evaluated the potential occupational exposures that would result from the operational sequence to transfer spent fuel assemblies from the IP3 SFP to the IP2 SFP. The evaluation concluded that the radiation dose to workers would be within the dose limits specified in 10 CFR 20.1201. The NRC staff reviewed the dose estimates for the transfer operations in its safety evaluation for the proposed action and concluded that the dose estimates for the operations activities are reasonable. Based on the above, there are no significant occupational dose impacts associated with the proposed action.

### Offsite Doses to Members of the Public

The licensee will maintain radiological controls in accordance with its radiation protection program throughout the spent fuel transfer operations. The licensee's evaluation of the potential dose to a member of the public at the boundary of the plant's controlled area during the proposed action shows that offsite doses would be within the public dose limit in 10 CFR 20.1301. Based on the above, the offsite radiation dose to members of the public would continue to be within NRC regulatory limits and, therefore, would not be significant.

### Accident Doses to Members of the Public

Various accidents were postulated, such as a dropped fuel assembly, extended time delays during transfer operations, a dropped shielded cask full of spent fuel, a fire involving the cask transporter, a tornado during transfer

operations, and a tipover of the shielded cask full of spent fuel. These accidents were analyzed by the licensee and the analyses were reviewed by NRC staff to assure that there is no undue hazard to the health and safety of the public. The licensee calculated the dose to a member of the public at the boundary of the plant's controlled area for accident conditions involving the spent fuel transfer operations. The licensee's analyses demonstrate that the dose to members of the public will be within the public dose limits in 10 CFR 20.1301. The NRC staff, in its safety evaluation, found the licensee's evaluation to be reasonable. Based on the above, the offsite radiation dose to members of the public in the event of a fuel transfer accident would continue to be within NRC regulatory limits and, therefore, would not be significant.

### Alternatives to the Proposed Action

As an alternative to the proposed action the licensee considered using a spent fuel cask which was already licensed as a transportation package under 10 CFR part 71. The licensee identified one cask which could be lifted by the existing IP3 crane, but it only had the capacity for a single fuel assembly. This would severely limit the rate of fuel transfer and would also increase the total radiation exposure to the workers involved with fuel movement. Using that cask would entail similar operations as using the STC, which holds up to 12 fuel assemblies, but the result would be almost 12 times as many trips from the IP3 FSB to the IP2 FSB.

The NRC staff also considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in the current environmental impacts. However, if the proposed action were not approved for IP2 and IP3, Entergy would have to consider installing an IP3 spent fuel cask handling crane with at least a 100-ton capacity to lift and handle its standard HI-TRAC fuel transfer cask. Such an action would require major upgrades to plant equipment and modifications to plant structures, as well as radiation doses to workers in the IP3 FSB during the construction process.

### Alternative Use of Resources

The action does not involve the use of any different resources than those previously considered in the Final Environmental Statement for IP2, dated September 30, 1972, ADAMS Accession Nos. ML072390276 and ML072390278, or the Final Environmental Statement for IP3, dated February 28, 1975,

ADAMS Accession Nos. ML072390284 and ML072390286.

### Agencies and Persons Consulted

In accordance with its stated policy, on February 17, 2012, the NRC staff consulted with the designated New York State official regarding the environmental impacts of the proposed action. The State official had no comments on the environmental impacts.

### Finding of No Significant Impact

On the basis of the environmental assessment, the NRC staff concludes that granting the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC staff has determined it is not necessary to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's application dated July 8, 2009, Agencywide Documents Access and Management System (ADAMS) Accession No. ML091940176, as supplemented by letters dated September 28, 2009, ADAMS Accession No. ML092950437; October 26, 2009, ADAMS Accession No. ML093020080; October 5, 2010, ADAMS Accession No. ML102910511; October 28, 2010, ADAMS Accession Nos. ML103080112 and ML103080113; July 28, 2011, ADAMS Accession No. ML11220A079; August 23, 2011, ADAMS Accession Nos. ML11243A174, ML11243A175; and ML11243A220; October 28, 2011, ADAMS Accession No. ML11327A045 and ML11327A046; December 15, 2011, ADAMS Accession No. ML12013A259; January 11, 2012, ADAMS Accession No. ML120400604; March 2, 2012, ADAMS Accession No. ML12074A027, April 23, 2012, ADAMS Accession No. ML12129A457, and May 7, 2012, ADAMS Accession No. ML121370318. Publicly available versions of the documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available documents created or received at the NRC are accessible electronically through the Agencywide Documents Access and Management System (ADAMS) in the NRC Electronic Library at <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone

at 1-800-397-4209 or 301-415-4737, or send an email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

**FOR FURTHER INFORMATION CONTACT:** John Boska, Office of Nuclear Reactor Regulation, Mail Stop 0-8C2, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, by telephone at 301-415-2901, or by email at [John.Boska@nrc.gov](mailto:John.Boska@nrc.gov).

Dated at Rockville, Maryland, this 5th day of July 2012.

For the Nuclear Regulatory Commission.

**John P. Boska,**

*Senior Project Manager, Plant Licensing Branch I-1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2012-17110 Filed 7-12-12; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[NRC-2012-0170]

### Aging Management Associated With Wall Thinning Due to Erosion Mechanisms

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Draft interim staff guidance; request for public comment.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC or the Commission) requests public comment on Draft License Renewal Interim Staff Guidance (LR-ISG), LR-ISG-2012-01, "Wall Thinning Due to Erosion Mechanisms." The draft LR-ISG proposes to revise an NRC staff-recommended aging management program (AMP) in NUREG-1801, Revision 2, "Generic Aging Lessons Learned (GALL) Report," and the NRC staff's aging management review procedure and acceptance criteria contained in NUREG-1800, Revision 2, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants" (SRP-LR) to address wall thinning due to various erosion mechanisms for piping and components within the scope of the Requirements for Renewal of Operating Licenses for Nuclear Power Plants. This LR-ISG provides changes to the recommendations in GALL Report, Revision 2, AMP XI.M17, "Flow-Accelerated Corrosion," based on the staff's review of several license renewal applications' flow-accelerated corrosion AMPs and stakeholder input.

**DATES:** Submit comments by August 27, 2012. Comments received after this date will be considered, if it is practical to do so, but the NRC staff is able to ensure

consideration only for comments received on or before this date.

**ADDRESSES:** You may access information and comment submissions related to this document, which the NRC possesses and are publicly available, by searching on <http://www.regulations.gov> under Docket ID NRC-2012-0170. You may submit comments by any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2012-0170. Address questions about NRC dockets to Carol Gallagher; telephone: 301-492-3668; email: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov).

- *Mail comments to:* Cindy Bladey, Chief, Rules, Announcements, and Directives Branch (RADB), Office of Administration, Mail Stop: TWB-05-B01M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

- *Fax comments to:* RADB at 301-492-3446.

For additional direction on accessing information and submitting comments, see "Accessing Information and Submitting Comments" in the **SUPPLEMENTARY INFORMATION** section of this document.

**FOR FURTHER INFORMATION CONTACT:** Mr. James Gavula, Division of License Renewal, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 630-829-9755; email: [James.Gavula@nrc.gov](mailto:James.Gavula@nrc.gov).

### SUPPLEMENTARY INFORMATION:

#### I. Accessing Information and Submitting Comments

##### A. Accessing Information

Please refer to Docket ID NRC-2012-0170 when contacting the NRC about the availability of information regarding this document. You may access information related to this document, which the NRC possesses and are publicly available, by any of the following methods:

- *Federal Rulemaking Web Site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2012-0170.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may access publicly available documents online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The draft