

directed by the Antarctic Conservation Act of 1978 (Pub. L. 95–541), as amended by the Antarctic Science, Tourism and Conservation Act of 1996, has developed regulations for the establishment of a permit system for various activities in Antarctica and designation of certain animals and certain geographic areas requiring special protection. The regulations establish such a permit system to designate Antarctic Specially Protected Areas.

The applications received are as follows:

#### Permit Application: 2012–006

1. *Applicant:* Jeff Bowman, University of Washington, Box 357940, Seattle, WA 98105–7940.

#### Activity for Which Permit Is Requested

Enter an Antarctic Specially Protected Area. The applicant plans to enter the Antarctic Specially Protect Area at Cape Royds, Ross Island (ASPA 121) to collect sea ice and seawater for microbial analysis.

#### Location

ASPA 121—Cape Royds, Ross island.

#### Dates

August 27, 2011 to November 2, 2011.

**Nadene G. Kennedy,**

*Permit Officer, Office of Polar Programs.*

[FR Doc. 2011–21296 Filed 8–19–11; 8:45 am]

**BILLING CODE 7555–01–P**

## NUCLEAR REGULATORY COMMISSION

[NRC–2010–0278]

### NUREG–1482, Revision 2, “Guidelines for Inservice Testing at Nuclear Power Plants, Draft Report for Comment”

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Announcement of issuance for public comment, availability.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) has issued for public comment a document entitled: NUREG–1482, Revision 2, “Guidelines for Inservice Testing at Nuclear Power Plants, Draft Report for Comment,” and subtitled “Inservice Testing of Pumps and Valves, and Inservice Examination and Testing of Dynamic Restraints (Snubbers) at Nuclear Power Plants”.

(Note that this document was submitted previously for public comments as draft NUREG–1946. Based on public comments, draft NUREG–1482 is being updated as Revision 2 and

is being issued to incorporate all comments received for draft report NUREG–1946).

**DATES:** Please submit comments by December 20, 2011. Comments received after this date will be considered if it is practical to do so, but the NRC staff is able to ensure consideration only for comments received on or before this date.

**ADDRESSES:** Please include Docket ID NRC–2010–0278 in the subject line of your comments. For additional instructions on submitting comments and instructions on accessing documents related to this action, see “Submitting Comments and Accessing Information” in the **SUPPLEMENTARY INFORMATION** section of this document. You may submit comments by any one of the following methods:

- *Federal Rulemaking Web Site:* Go to <http://www.regulations.gov> and search for documents filed under Docket ID NRC–2010–0278. Address questions about NRC dockets to Carol Gallagher, telephone: 301–492–3668; e-mail: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov).

- *Mail comments to:* Cindy Bladey, Chief, Rules, Announcements, and Directives Branch (RADB), Office of Administration, Mail Stop: TWB–05–B01M, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

- *Fax comments to:* RADB at 301–492–3446.

#### SUPPLEMENTARY INFORMATION:

#### Submitting Comments and Accessing Information

Comments submitted in writing or in electronic form will be posted on the NRC Web site and on the Federal rulemaking Web site, <http://www.regulations.gov>. Because your comments will not be edited to remove any identifying or contact information, the NRC cautions you against including any information in your submission that you do not want to be publicly disclosed.

The NRC requests that any party soliciting or aggregating comments received from other persons for submission to the NRC inform those persons that the NRC will not edit their comments to remove any identifying or contact information, and therefore, they should not include any information in their comments that they do not want publicly disclosed.

You can access publicly available documents related to this document using the following methods:

- *NRC’s Public Document Room (PDR):* The public may examine and have copied, for a fee, publicly available

documents at the NRC’s PDR, O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

- *NRC’s Agencywide Documents Access and Management System (ADAMS):* Publicly available documents created or received at the NRC are available online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. From this page, the public can gain entry into ADAMS, which provides text and image files of the NRC’s public documents. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC’s PDR reference staff at 1–800–397–4209, 301–415–4737, or by e-mail to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The NUREG–1482, Revision 2, “Guidelines for Inservice Testing at Nuclear Power Plants, Draft Report for Comment,” and subtitled “Inservice Testing of Pumps and Valves, and Inservice Examination and Testing of Dynamic Restraints (Snubbers) at Nuclear Power Plants” is available electronically under ADAMS Accession Number ML112231412.

- *Federal Rulemaking Web Site:* Public comments and supporting materials related to this notice can be found at <http://www.regulations.gov> by searching on Docket ID NRC–2010–0278.

#### FOR FURTHER INFORMATION CONTACT:

Gurjendra S. Bedi, Division of Component Integrity, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone: 301–415–1393, e-mail: [Gurjendra.Bedi@nrc.gov](mailto:Gurjendra.Bedi@nrc.gov).

NUREG–1482, Revision 2, “Guidelines for Inservice Testing at Nuclear Power Plants, Draft Report for Comment” provides updated information on applicable regulations for testing of pumps and valves, and inservice examination and testing of snubbers. The information in NUREG–1482, “Guidelines for Inservice Testing at Nuclear Plants,” Revision 0, issued April 1995, and Revision 1, issued January 2005, has described these topics in the past.

This NUREG report replaces Revision 0 and Revision 1 of NUREG–1482, and is applicable, unless stated otherwise, to all editions and addenda of the American Society of Mechanical Engineers (ASME) Code of Operation and Maintenance of Nuclear Power Plants (OM Code), which Title 10 of the Code of Federal Regulations (10 CFR) 50.55a(b) incorporates by reference (76 FR 36232–36279), dated June 21, 2011. Note that the draft NUREG–1482, Revision 2, is a substantial re-write from issuance of draft NUREG–1946 (ADAMS

Accession Number: ML102100236). This draft NUREG–1482, Revision 2, incorporates all the public comments received for draft NUREG–1946, because based on public comments, NUREG–1482, Revision 1, is revised and updated instead of issuing the new NUREG–1946. The NRC staff evaluation and resolution of public comments for draft NUREG–1946, including Inservice Testing Owner Group comments, are documented in ADAMS Accession Number: ML112092872. Most of the draft NUREG–1946 included in the main text of draft NUREG–1482, Revision 2, Appendix A, to this NUREG–1482, Revision 2, contains guidance provided in Revision 1 to NUREG–1482 for pumps and valves that has been updated for the development of inservice testing programs at nuclear power plants. Appendix B to this NUREG contains guidance related to inservice examination and testing of dynamic restraints (snubbers), which is included for the first time in the draft NUREG–1482, Revision 2.

The guidelines and recommendations provided in this NUREG and its Appendices A and B do not supersede the regulatory requirements specified in 10 CFR 50.55a. Further, this NUREG does not authorize the use of alternatives to, or grant relief from, the ASME Code requirements for inservice testing of pumps and valves, or inservice examination and testing of dynamic restraints (snubbers), incorporated by reference in 10 CFR 50.55a. In addition, the NUREG discusses other inservice test program topics such as the NRC process for review of the OM Code, conditions on the use of the OM Code, and interpretations of the OM Code.

Dated at Rockville, Maryland, this 11th day of August 2011.

For the Nuclear Regulatory Commission.

**Anthony C. McMurtray,**

*Chief, Component Performance and Testing Branch, Division of Component Integrity, Office of Nuclear Reactor Regulation.*

[FR Doc. 2011–21357 Filed 8–19–11; 8:45 am]

**BILLING CODE 7590–01–P**

## **NUCLEAR REGULATORY COMMISSION**

**[NRC–2011–0188; Docket No. 50–315]**

### **Indiana Michigan Power Company, Donald C. Cook Nuclear Plant, Unit 1; Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (NRC or the Commission) is considering issuance of an exemption

and an amendment to Renewed Facility Operating License No. NPF–58 issued to Indiana Michigan Power Company (the licensee), for operation of Donald C. Cook Nuclear Plant, Unit 1 (DCCNP–1), located in Berrien County, Michigan, in accordance with Title 10 of the Code of Federal Regulations (10 CFR) part 50, § 50.90. In accordance with 10 CFR 51.21, the NRC performed an environmental assessment documenting its findings. The NRC concluded that the proposed actions would have no significant environmental impact.

#### **Environmental Assessment**

##### *Identification of the Proposed Actions*

The proposed actions would issue an exemption from 10 CFR 50.46 regarding fuel cladding material, and revise the Technical Specifications document, which is part of the Renewed Facility Operating Licenses, to permit use of Optimized ZIRLO™ fuel to a peak rod average burnup limit of 62 gigawatt-days per metric ton uranium (GWD/MTU).

The proposed actions are in accordance with the licensee's application dated December 16, 2010.

##### *The Need for the Proposed Actions*

The proposed actions to issue an exemption to the fuel cladding requirement of 10 CFR 50.46, and to amend the Technical Specifications to permit use of Optimized ZIRLO™ fuel to a peak rod average burnup limit of 62 GWD/MTU would allow for more effective fuel management. If the exemption and amendment are not approved, the licensee will not be provided the opportunity to use Optimized ZIRLO™ fuel design with a peak rod average burnup as high as 62 GWD/MTU; the licensee would thus lose fuel management flexibility.

##### *Environmental Impacts of the Proposed Actions*

In this environmental assessment regarding the impacts of the use of Optimized ZIRLO™ fuel with the possible burnup up to 62 GWD/MTU, the Commission is relying on the results of the updated study conducted for the NRC by the Pacific Northwest National Laboratory (PNNL), entitled "Environmental Effects of Extending Fuel Burnup Above 60 GWD/MTU" (NUREG/CR–6703, PNNL–13257, January 2001). Environmental impacts of high burnup fuel up to 75 GWD/MTU were evaluated in the study, but some aspects of the review were limited to evaluating the impacts of the extended burnup up to 62 GWD/MTU, because of the need for additional data on the effect

of extended burnup on gap release fractions. All the aspects of the fuel-cycle were considered during the study, from mining, milling, conversion, enrichment and fabrication through normal reactor operation, transportation, waste management, and storage of spent fuel.

The amendment and exemption would allow DCCNP–1 to use Optimized ZIRLO™ fuel up to a burnup limit of 62 GWD/MTU. The NRC staff has completed its evaluation of the proposed actions and concludes that such changes would not adversely affect plant safety, and would have no adverse effect on the probability of any accident. For the accidents that involve damage or melting of the fuel in the reactor core, fuel rod integrity has been shown to be unaffected by extended burnup under consideration; therefore, the probability of an accident will not be affected by fuel burnup to 62 GWD/MTU. For the accidents in which the reactor core remains intact, the increased burnup may slightly change the mix of fission products that could be released in the event of a serious accident, but because the radionuclides contributing most to the dose are short-lived, increased burnup would not have an effect on the consequences of a serious accident beyond the consequences of previously evaluated accident scenarios. Thus, there will be no significant increase in projected dose consequences of postulated accidents associated with fuel burnup up to 62 GWD/MTU, and doses will remain well below regulatory limits.

Regulatory limits on radiological effluent releases are independent of burnup. The requirements of 10 CFR part 20, 10 CFR 50.36a, and Appendix I to 10 CFR part 50 ensure that routine releases of gaseous, liquid or solid radiological effluents to unrestricted areas is kept "As Low As is Reasonably Achievable." Therefore, the NRC staff concludes that during routine operations, there would be no significant increase in the amount of gaseous radiological effluents released into the environment as a result of the proposed actions, nor will there be a significant increase in the amount of liquid radiological effluents or solid radiological effluents released into the environment.

The proposed actions will not change normal plant operating conditions (*i.e.*, no changes are expected in the fuel handling, operational, or storing processes). The fuel storage and handling, radioactive waste, and other systems which may contain radioactivity are designed to assure adequate safety under normal