ACTION: Notice of Granting of Request to Extend the Comment Period of Draft Regulatory Guides, DG–1186 and DG–4013.

FOR FURTHER INFORMATION CONTACT: Steve Garry, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone: (301) 415–2766 or e-mail to *Steve.Garry@nrc.gov*.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) issued for public comment DG–1186, ''Measuring, Evaluating, and Reporting Radioactive Materials in Liquid and Gaseous Effluents and Solid Wastes,'' which was published in the **Federal Register**, 73 FR 65705, on November 4, 2008. DG–1186 is proposed Revision 2 of Regulatory Guide 1.21.

NRC also issued for public comment DG–4013, ''Environmental Monitoring for Nuclear Power Plants,'' which was published in the **Federal Register**, 73 FR 66685, on November 10, 2008. DG–4013 is proposed Revision 2 of Regulatory Guide 4.1.

The DGs are in the agency's ''Regulatory Guide'' series. This series has been developed to describe and make available to the public such information as methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

II. Further Information

The NRC staff requested receipt of comments on DG–1186 by December 30, 2008, and the receipt of comments on DG–4013 by January 9, 2009. By this action, the NRC staff is extending the comment period until January 30, 2009, for DG–1186 and DG–4013. Comments received after January 30, 2009 for DG–1186 and DG–4013, would be considered if practical to do so, but the NRC is able to ensure consideration only for comments received on or before January 30, 2009. Although a time limit is given, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

III. Request To Extend the Comment Period

Basis for the Request

The NRC received the following extension request:

In a letter, the Nuclear Energy Institute requested that the public review and comment period on DG–1186 and DG–4013 be extended to January 30, 2009. This will allow the public an opportunity to formulate comments because there is an NRC public meeting on the two draft guides scheduled for January 15, 2009.

Response to Request

The request for an extension to the comment period is approved until January 30, 2009.

Requests for technical information about DG–1186 and DG–4013 may be directed to the NRC contact, Steve Garry at (301) 415–2766 or e-mail to *Steve.Garry@nrc.gov*.

Electronic copies of DG–1186 are available through the NRC's public Web site under Draft Regulatory Guides in the ''Regulatory Guides'' collection of the NRC's Electronic Reading Room at *http://www.nrc.gov/reading-rm/doc-collections/*. Electronic copies are also available in ADAMS (*http://www.nrc.gov/reading-rm/adams.html*), under Accession No. ML080660617.

Electronic copies of DG–4013 are also available through the NRC's public Web site under Draft Regulatory Guides in the ''Regulatory Guides'' collection of the NRC's Electronic Reading Room at *http://www.nrc.gov/reading-rm/doc-collections/*. Electronic copies are also available in ADAMS (*http://www.nrc.gov/reading-rm/adams.html*), under Accession No. ML080660608.

In addition, regulatory guides are available for inspection at the NRC's Public Document Room (PDR), which is located at 11555 Rockville Pike, Rockville, Maryland. The PDR's mailing address is USNRC PDR, Washington, DC 20555–0001. The PDR can also be reached by telephone at (301) 415–4737 or (800) 397–4205, by fax at (301) 415–3548, and by e-mail to *pdr.resource@nrc.gov*.

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Dated at Rockville, Maryland, this 17th day of December, 2008.

For the Nuclear Regulatory Commission.

Andrea D. Valentin,

Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50–400]

Carolina Power & Light Company; Shearon Harris Nuclear Power Plant, Unit 1; Notice of Issuance of Renewed Facility Operating License No. NPF–63 for an Additional 20-Year Period; Record of Decision

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission or NRC) has issued Renewed Facility Operating License No. NPF–63 to the Carolina Power & Light Company (the licensee), now doing business as Progress Energy Carolinas, Inc., the operator of the Shearon Harris Nuclear Power Plant (HNP), Unit 1. Renewed Facility Operating License No. NPF–63 authorizes operation of HNP by the licensee at reactor core power levels not in excess of 2900 megawatts thermal (900 megawatts electric), in accordance with the provisions of the HNP renewed license and its Technical Specifications.

This notice also serves as the record of decision for the renewal of Facility Operating License No. NPF–63 for HNP, consistent with Title 10 of the *Code of Federal Regulations* (10 CFR) Section 51.103. As discussed in the Final Supplemental Environmental Impact Statement (FSEIS) for HNP, dated August 2008, the Commission considered a range of reasonable alternatives that included generation from coal, natural gas, oil, wind, solar, hydropower, geothermal, wood waste, municipal solid waste, other biomass-derived fuels, delayed retirement, utility-sponsored conservation, a combination of alternatives, and a no-action alternative. The factors considered in the record of decision can be found in the supplemental environmental impact statement for License Renewal, Supplement 33 regarding HNP, Unit 1.

HNP is a pressurized-water reactor designed by the Westinghouse Electric Corporation that is located in Wake County, North Carolina, approximately twenty miles southwest of Raleigh, North Carolina. The application for the renewed license complied with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. As required by the Act and the Commission's regulations in 10 CFR Chapter I, the Commission has made appropriate findings, which are set forth in the license.

Prior public notice of the action involving the proposed issuance of the renewed license and of an opportunity for a hearing regarding the proposed

issuance of the renewed license was published in the **Federal Register** on March 20, 2007 (72 FR 13139). For further details with respect to this action, see (1) The Carolina Power & Light Company, license renewal application for HNP, dated November 14, 2006, as supplemented by letters dated through July 21, 2008; (2) the Commission's Safety Evaluation Report (NUREG–1916 Volumes 1 and 2), published in December 2008; (3) the licensee's updated safety analysis report; and (4) the Commission's FSEIS (NUREG–1437, Supplement 33), published in August 2008. These documents are available at the NRC Public Document Room, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852, and can be viewed from the NRC Public Electronic Reading Room at *http://www.nrc.gov/reading-rm/adams.html*.

Copies of the Renewed Facility Operating License No. NPF–63 may be obtained by writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Director, Division of License Renewal. Copies of the HNP, Unit 1, Safety Evaluation Report (NUREG–1916 Volumes 1 and 2) and the FSEIS (NUREG–1437, Supplement 33) may be purchased from the National Technical Information Service, U.S. Department of Commerce, Springfield, Virginia 22161–0002 (*http://www.ntis.gov*), 703–605–6000, or the Superintendent of Documents, U.S. Government Printing Office, P.O. Box 371954, Pittsburgh, Pennsylvania 15250–7954, (*http://www.gpoaccess.gov*), 202–512–1800. All orders should clearly identify the NRC publication number and the requester's Government Printing Office deposit account number or a VISA or MasterCard number and expiration date.

Dated at Rockville, Maryland, this 17th day of December 2008.

For the Nuclear Regulatory Commission.

Brian E. Holian,

Director, Division of License Renewal, Office of Nuclear Reactor Regulation.

[FR Doc. E8–30477 Filed 12–22–08; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50–483]

Union Electric Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF–30 issued to Union Electric Company (the licensee) for operation of the Callaway Plant, Unit 1 located in Callaway County, Missouri.

By letter dated October 31, 2008, the Nuclear Regulatory Commission issued Amendment No. 186, to Callaway Plant, Unit 1, Facility Operating License No. NPF–30. The amendment allowed a one-time extension of the allowed outage time (completion time) for each of the two essential service water (ESW) trains (ESW Train A and Train B) from 72 hours to 14 days. The extended completion time was requested to support planned replacement of the underground carbon steel piping with new high density polyethylene (HDPE) piping for ESW Train A and ESW Train B during plant operation. The amendment was issued with a requirement to complete the replacement of carbon steel piping with HDPE for both ESW trains by December 31, 2008. By its application dated December 1, 2008, the licensee informed NRC that it had experienced significant delays in completing the replacement of underground piping/conduit of ESW Train A, due, in part, to underground obstructions during excavation, a longer refueling outage (Refuel 16) than anticipated, a forced outage at the beginning of Cycle 17, switchyard maintenance, and other equipment and personnel issues. Therefore, the licensee proposed a change in plant modification dates for replacement of ESW Train B carbon steel piping. The present plan calls for completing the modification of ESW Train A by December 31, 2008, as approved by Amendment No. 186. Consequently, the proposed amendment would extend the implementation date for completion of replacement of carbon steel piping for ESW Train B from December 31, 2008, to April 30, 2009.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in Title 10 of the Code of Federal Regulations (10 CFR), Section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

Overall protection system performance will remain within the bounds of the previously performed accident analyses since no hardware changes are proposed to the protection systems. The same reactor trip system (RTS) and engineered safety feature actuation system (ESFAS) instrumentation will continue to be used. The protection systems will continue to function in a manner consistent with the plant design basis. The use of polyethylene (PE) piping in the ESW system will result in improved system performance and enhanced system reliability, and will provide an acceptable level of quality and safety. There will be no changes to the essential service water (ESW) system or ultimate heat sink (UHS) surveillance and operating limits.

The proposed changes will not adversely affect accident initiators or precursors nor alter the design assumptions, conditions, and configuration of the facility or the manner in which the plant is operated and maintained. The proposed changes will not alter or prevent the ability of structures, systems, and components (SSCs) from performing their intended functions to mitigate the consequences of an initiating event within the assumed acceptance limits.

The proposed changes do not affect the way in which safety-related systems perform their functions.

All accident analysis acceptance criteria will continue to be met with the proposed changes. The proposed changes will not affect the source term, containment isolation, or radiological release assumptions used in evaluating the radiological consequences of an accident previously evaluated. The proposed changes will not alter any assumptions or change any mitigation actions in the radiological consequence evaluations in the FSAR. The applicable radiological dose acceptance criteria will continue to be met.

Therefore, the proposed changes do not involve a significant increase in the