

The NRC staff will hold a public meeting to present an overview of the draft plant-specific supplement to the GEIS and to accept public comments on the document. The public meeting will be held on May 28, 2008, at The Eagles Building, 107 South Market St., Berwick, PA 18603. There will be two sessions to accommodate interested parties. The first session will convene at 1:30 p.m. and will continue until 4:30 p.m., as necessary. The second session will convene at 7 p.m. with a repeat of the overview portions of the meeting and will continue until 10 p.m., as necessary. Both meetings will be transcribed and will include: (1) A presentation of the contents of the draft plant-specific supplement to the GEIS, and (2) the opportunity for interested government agencies, organizations, and individuals to provide comments on the draft report. Additionally, the NRC staff will host informal discussions one hour prior to the start of each session at the same location. No comments on the draft supplement to the GEIS will be accepted during the informal discussions. To be considered, comments must be provided either at the transcribed public meeting or in writing. Persons may pre-register to attend or present oral comments at the meeting by contacting Mr. Drew Stuyvenberg, the NRC Environmental Project Manager at 1-800-368-5642, extension 4006, or by e-mail at SusquehannaEIS@nrc.gov no later than May 23, 2008. Members of the public may also register to provide oral comments within 15 minutes of the start of each session. Individual, oral comments may be limited by the time available, depending on the number of persons who register. If special equipment or accommodations are needed to attend or present information at the public meeting, the need should be brought to Mr. Stuyvenberg's attention no later than May 16, 2008, to provide the NRC staff adequate notice to determine whether the request can be accommodated.

FOR FURTHER INFORMATION, CONTACT: Mr. Drew Stuyvenberg, Renewal Projects Branch 1, Division of License Renewal, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Mail Stop O-11F1, Washington, DC 20555-0001. Mr. Stuyvenberg may be contacted at the aforementioned telephone number or e-mail address.

Dated at Rockville, Maryland, this 25th day of April, 2008.

For the Nuclear Regulatory Commission.

Louise Lund,

Branch Chief, Renewal Projects Branch 1, Division of License Renewal, Office of Nuclear Reactor Regulation.

[FR Doc. E8-9593 Filed 4-30-08; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Sunshine Federal Register Notice

AGENCY HOLDING THE MEETINGS: Nuclear Regulatory Commission.

DATE: Week of April 28, 2008.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and Closed.

ADDITIONAL MATTERS TO BE CONSIDERED:

Week of April 28, 2008

Wednesday, April 30, 2008

12:55 p.m., Affirmation Session (Public Meeting) (Tentative).

- a. Indian Point Nuclear Generating Units 2 and 3, Docket Nos. 50-247-LR and 50-286-LR; WestCAN "Petition for Review" of Board Order denying oral argument on contention admissibility in the Indian Point license renewal proceeding (Tentative).
- b. Pacific Gas and Electric Co. Diablo Canyon ISFSI, San Luis Obispo Mothers for Peace's Motion for Reconsideration of CLI-08-05 & San Luis Obispo Mothers for Peace's Request for Admission of Late-Filed Contention 6 Re Diablo Canyon Environmental Assessment (Tentative).

This meeting will be webcast live at the Web address—<http://www.nrc.gov>.

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*The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (recording)—(301) 415-1292. Contact person for more information: Michelle Schroll, (301) 415-1662.

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Additional Information

By a vote of 4-0 on April 24 and 25, 2008, the Commission determined pursuant to U.S.C. 552b(e) and § 9.107(a) of the Commission's rules that "Affirmation of: a. Indian Point Nuclear Generating Units 2 and 3, Docket Nos. 50-247-LR and 50-286-LR; WestCAN "Petition for Review" of Board Order denying oral argument on contention admissibility in the Indian Point license renewal proceeding (Tentative) and b.

Pacific Gas and Electric Co. Diablo Canyon ISFSI, San Luis Obispo Mothers for Peace's Motion for Reconsideration of CLI-08-05 & San Luis Obispo Mothers for Peace's Request for Admission of Late-Filed Contention 6 Re Diablo Canyon Environmental Assessment (Tentative)" be held April 30, 2008, and on less than one week's notice to the public.

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The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/about-nrc/policy-making/schedule.html>.

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The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (e.g. braille, large print), please notify the NRC's Disability Program Coordinator, Rohn Brown, at 301-415-2279, TDD: 301-415-2100, or by e-mail at Rohn.Brown@nrc.gov. Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

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This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to the distribution, please contact the Office of the Secretary, Washington, DC 20555 (301-415-1969). In addition, distribution of this meeting notice over the Internet system is available. If you are interested in receiving this Commission meeting schedule electronically, please send an electronic message to dkw@nrc.gov.

Dated: April 28, 2008.

R. Michelle Schroll,

Office of the Secretary.

[FR Doc. 08-1204 Filed 4-29-08; 10:58 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Proposed Generic Communication; Fatigue Analysis of Nuclear Power Plant Components

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of opportunity for public comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is proposing to issue a regulatory issue summary (RIS) to inform licensees of an analysis

methodology used to demonstrate compliance with the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) fatigue acceptance criteria that could be nonconservative if not correctly applied.

This **Federal Register** notice is available through the NRC's Agencywide Documents Access and Management System (ADAMS) under accession number ML081080562.

DATES: Comment period expires June 16, 2008. Comments submitted after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except for comments received on or before this date.

ADDRESSES: Submit written comments to the Chief, Rulemaking, Directives and Editing Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Mail Stop T6-D59, Washington, DC 20555-0001, and cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to NRC Headquarters, 11545 Rockville Pike (Room T-6D59), Rockville, Maryland, between 7:30 a.m. and 4:15 p.m. on Federal workdays.

FOR FURTHER INFORMATION, CONTACT: John R. Fair at 301-415-2759 or by e-mail at John.Fair@nrc.gov.

SUPPLEMENTARY INFORMATION:

NRC Regulatory Issue Summary 2008-XX

Fatigue Analysis of Nuclear Power Plant Components

Addressees

All holders of operating licenses for nuclear power reactors, except those who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel.

Intent

The U.S. Nuclear Regulatory Commission (NRC) is issuing this regulatory issue summary (RIS) to inform licensees of an analysis methodology used to demonstrate compliance with the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) fatigue acceptance criteria that could be nonconservative if not correctly applied.

Background Information

Title 10 of the Code of Federal Regulations (10 CFR) Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants," requires that applicants for license renewal perform an evaluation of time-

limited aging analyses relevant to structures, systems, and components within the scope of license renewal. The fatigue analysis of the reactor coolant pressure boundary components is an issue that involves time-limited assumptions. In addition, the staff has provided guidance in NUREG-1800, Rev. 1, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants," issued September 2005. NUREG-1800, Rev. 1, specifies that the effects of the reactor water environment on fatigue life be evaluated for a sample of components to provide assurance that cracking because of fatigue will not occur during the period of extended operation. Since the reactor water environment has a significant impact on the fatigue life of components, many license renewal applicants have performed supplemental detailed analyses to demonstrate acceptable fatigue life for these components.

10 CFR 50.55a, "Codes and Standards," specifies the ASME Code requirements for operating reactors. Some operating facilities may have performed supplemental detailed analysis of components because of new loading conditions identified after the plant began operation.

Summary of Issue

The staff identified a concern regarding the methodology used by some license renewal applicants to demonstrate the ability of nuclear power plant components to withstand the cyclic loads associated with plant transient operations for the period of extended operation. This particular analysis methodology involves the use of the Green's function to calculate the fatigue usage during plant transient operations such as startups and shutdowns.

The Green's function approach involves performing a detailed stress analysis of a component to calculate its response to a step change in temperature. This detailed analysis is used to establish an influence function, which is subsequently used to calculate the stresses caused by the actual plant temperature transients. This methodology has been used to perform fatigue calculations and as input for on-line fatigue monitoring programs. The Green's function methodology is not in question. The concern involves a simplified input for applying the Green's function in which only one value of stress is used for the evaluation of the actual plant transients. The detailed stress analysis requires consideration of six stress components, as discussed in ASME Code, Section III,

Subsection NB, Subarticle NB-3200. Simplification of the analysis to consider only one value of the stress may provide acceptable results for some applications; however, it also requires a great deal of judgment by the analyst to ensure that the simplification still provides a conservative result.

The staff has requested that recent license renewal applicants that have used this simplified Green's function methodology perform confirmatory analyses to demonstrate that the simplified Green's function analyses provide acceptable results. The confirmatory analyses retain all six stress components. To date, the confirmatory analysis of one component, a boiling-water reactor feedwater nozzle, indicated that the simplified input for the Green's function did not produce conservative results in the nozzle bore area when compared to the detailed analysis. However, the confirmatory analysis still demonstrated that the nozzle had acceptable fatigue usage.

Licensees may have also used the simplified Green's function methodology in operating plant fatigue evaluations for the current license term. For plants with renewed licenses, the staff is considering additional regulatory actions if the simplified Green's function methodology was used.

Backfit Discussion

This RIS informs addressees of a potential nonconservative calculation methodology and reminds them that the ASME Code fatigue analysis should be performed properly. For license renewal, metal fatigue is evaluated as a time-limited aging analysis in accordance with 10 CFR 54.21(c). The associated staff review guidance appears in Section 4.3, "Metal Fatigue Analysis," of NUREG-1800, Rev. 1. For operating reactors, the ASME Code requirements appear in 10 CFR 50.55a. This RIS does not impose a new or different regulatory staff position. It requires no action or written response and, therefore, is not a backfit under 10 CFR 50.109, "Backfitting." Consequently, the NRC staff did not perform a backfit analysis.

Federal Register Notification

To be done after the public comment period.

Congressional Review Act

The NRC has determined that this RIS is not a rule as designated by the Congressional Review Act (5 U.S.C. 801-808) and, therefore, is not subject to the Act.

Paperwork Reduction Act Statement

This RIS does not contain information collection requirements that are subject to the requirements of the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 *et seq.*).

Public Protection Notification

The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

Contact

Please direct any questions about this matter to the technical contacts listed below: Michael J. Case, Director, Division of Policy and Rulemaking, Office of Nuclear Reactor Regulation.

Technical Contacts: Kenneth C. Chang, NRR, E-mail: Kenneth.Chang@nrc.gov, 301-415-1913. John R. Fair, NRR, 301-415-2759, E-mail: John.Fair@nrc.gov.

Note: The NRC's generic communications may be found on the NRC public Web site, <http://www.nrc.gov>, under Electronic Reading Room/Document Collections.

End of Draft Regulatory Issue Summary

Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/NRC/ADAMS/index.html>. If you do not have access to ADAMS or if you have problems in accessing the documents in ADAMS, contact the NRC Public Document Room (PDR) reference staff at 1-800-397-4209 or 301-415-4737 or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 23rd day of April 2008.

For the Nuclear Regulatory Commission.

Martin C. Murphy,

Chief, Generic Communications Branch, Division of Policy and Rulemaking, Office of Nuclear Reactor Regulation.

[FR Doc. E8-9451 Filed 4-30-08; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-247]

Entergy Nuclear Indian Point 2, LLC; Entergy Nuclear Operations, Inc.; Notice of Withdrawal of Application for Amendment to Facility Operating License No. DPR-26 Indian Point Nuclear Generating Unit No. 2

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Entergy Nuclear Operations, Inc. (the licensee) to withdraw its March 22, 2007, application for proposed amendment to Facility Operating License No. DPR-26 for Indian Point Nuclear Generating Unit No. 2, located in Westchester County, New York.

The proposed amendment would have revised the test acceptance criteria for the emergency diesel generators endurance test.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on April 10, 2007 (72 FR 17947). However, by letter dated April 10, 2008, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated March 22, 2007, and the licensee's letter dated April 10, 2008, which withdrew the application for a license amendment. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, or 301-415-4737 or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 24th day of April 2008.

For the Nuclear Regulatory Commission.

John P. Boska,

Senior Project Manager, Plant Licensing Branch I-1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. E8-9586 Filed 4-30-08; 8:45 am]

BILLING CODE 7590-01-P

POSTAL REGULATORY COMMISSION**Sunshine Act Meetings**

NAME OF AGENCY: Postal Regulatory Commission.

TIME AND DATE: Tuesday, April 29, 2008 at 9 a.m.

PLACE: Commission conference room, 901 New York Avenue, NW., Suite 200, Washington, DC 20268-0001.

STATUS: Closed.

MATTERS TO BE CONSIDERED: Docket No. MC2008-1—consideration of motion to compel.

FOR FURTHER INFORMATION CONTACT: Stephen L. Sharfman, General Counsel, Postal Regulatory Commission, 901 New York Avenue, NW., Suite 200, Washington, DC 20268-0001, 202-789-6818.

Dated: April 28, 2008.

Steven W. Williams,
Secretary.

[FR Doc. 08-1205 Filed 4-29-08; 11:38 am]

BILLING CODE 7710-FW-P

POSTAL SERVICE**Sunshine Act Meeting**

DATE AND TIME: Tuesday, May 6, 2008, at 11:30 a.m.; and Wednesday, May 7, 2008, at 8:30 a.m. and 10:30 a.m.

PLACE: Washington, DC, at U.S. Postal Service Headquarters, 475 L'Enfant Plaza, SW., in the Benjamin Franklin Room.

STATUS: May 6—11:30 a.m.—Closed; May 7—8:30 a.m.—Open; May 7—10:30 a.m.—Closed.

MATTERS TO BE CONSIDERED

Tuesday, May 6 at 11:30 a.m. (Closed)

1. Strategic Issues.
2. Product Pricing.
3. Financial Update.
4. Personnel Matters and Compensation Issues.
5. Governors' Executive Session—Discussion of prior agenda items and Board Governance.

Wednesday, May 7 at 8:30 a.m. (Open)

1. Minutes of the Previous Meetings, April 1-2, and April 14, 2008.
2. Remarks of the Chairman of the Board.
3. Remarks of the Postmaster General and CEO.
4. Committee Reports.

Wednesday, May 7 at 8:30 a.m. (Open)
[continued]

5. Quarterly Report on Service Performance.