Specifically, the law requires that the Federal government create and maintain a single searchable Web site, accessible by the public at no cost to access, that includes specific data elements about most federal awards. Understanding this new legislation is extremely important to all GPC stakeholders because ultimately this law will apply to most awardees and sub-awardees, and federal agencies will require awardees to provide much of the needed data as a condition of receiving federal financial assistance.

Secondarily, the purpose of the webcast meeting is to inform stakeholders about the GPC's long-term planning and prioritization efforts and to receive input from stakeholders to inform a draft plan that will include both GPC's mission and vision, as well as a listing of GPC priorities as they relate to ongoing activities connected to the FGSI.

Meeting structure and agenda: The June 19 Webcast meeting will have the following structure and agenda:

(1) Welcome by the host agency;

(2) Overview of the FFATA by the Chair of the GPC;

(3) Overview of the GPC's long-term planning and proposed priorities by the Chair of the GPC; and

(4) Participants' discussion, questions and comments.

Background: Background about the FGSI is set forth in the **Federal Register** published on September 13, 2006 (71 FR 54098).

Dated: May 17, 2007.

Thomas Cooley,

Chair, Grants Policy Committee of the U.S. Chief Financial Officer Council. [FR Doc. E7–9839 Filed 5–22–07; 8:45 am] BILLING CODE 7555–01–P

NUCLEAR REGULATORY COMMISSION

Notice of Public Meeting for Fuel Cycle Facilities

AGENCY: Nuclear Regulatory Commission. **ACTION:** Public Meeting Notice.

FOR FURTHER INFORMATION CONTACT:

James Smith, Project Manager, Technical Support Section, Division of Fuel Cycle Safety and Safeguards, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20005– 0001. *Telephone:* (301) 415–6459; *fax number:* (301) 415–5370; *e-mail: jas4@nrc.gov.*

SUPPLEMENTARY INFORMATION:

I. Introduction

The Nuclear Regulatory Commission (NRC) is hosting a workshop to discuss issues of interest pertaining to the regulation of NRC-regulated fuel cycle facilities. The purpose of the workshop is to discuss various issues of the regulatory program related to the update of 10 CFR part 70. The specific issues to be discussed are 10 CFR part 70, Appendix A reportability of incidents, digital control systems, enforcement policy revisions, uranium solubility issues.

The workshop will be held in Rockville, Maryland, at the NRC's Executive Boulevard Building, located at 6003 Executive Boulevard and will be open to the public. We are expecting that NRC staff, licensees and certificate holders, and other interested parties and stakeholders will be making presentations on these issues of interest, with opportunity for followup discussion on each subject.

II. Dates and Location

Date: June 14, 2007. 9 a.m.–5:30 p.m. U.S. Nuclear Regulatory Commission, Executive Boulevard Building, 6003 Executive Boulevard, Rockville, MD 20852.

III. Contact

James Smith, Project Manager, Office of Nuclear Material Safety and Safeguards, Division of Fuel Cycle Safety and Safeguards, Special Projects Branch, *Mail Stop:* T8F42, 301–415– 6459, *Fax:* 301–415–5370, *e-mail: jas4@nrc.gov.*

IV. Further Information

The document related to this action is available electronically at the NRC's Electronic Reading Room at http:// www.nrc.gov/reading-rm/adams.html. From this site, you can access the NRC's Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. The ADAMS accession number for the document related to this notice is provided in the following table. If you do not have access to ADAMS or if there are problems in accessing the document located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to *pdr@nrc.gov*.

Dated at Rockville, Maryland, this 16th day of May 2007.

For the Nuclear Regulatory Commission. Margie Kotzolas,

Acting Chief, Technical Support Branch, Special Projects and Technical Support Directorate, Division of Fuel Cycle Safety and Safeguards, Office of Nuclear Materials Safety and Safeguards.

[FR Doc. E7–9923 Filed 5–22–07; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Notice of Availability of Model Safety Evaluation and Model License Amendment Request on Technical Specification Improvement Regarding Use of the Improved Banked Position Withdrawal Sequence for General Electric Boiling Water Reactors Using the Consolidated Line Item Improvement Process

AGENCY: Nuclear Regulatory Commission. ACTION: Request for comment.

SUMMARY: Notice is hereby given that the staff of the U.S. Nuclear Regulatory Commission (NRC) has prepared a model license amendment request (LAR), model safety evaluation (SE), and model proposed no significant hazards consideration (NSHC) determination related to changes to Standard Technical Specification (STS) 3.1.6, "Rod Pattern Control," and STS 3.3.2.1, "Control Rod Block Instrumentation" for NUREG-1433 and NUREG-1434. The proposed changes would revise the Technical Specifications (TS) and Bases for STS 3.1.6, "Rod Pattern Control," and STS 3.3.2.1, "Control Rod Block Instrumentation" to allow licensees to use an improved control rod banked position withdrawal sequence (BPWS) when performing a reactor shutdown. In addition, the proposed changes would add a footnote to Table 3.3.2.1-1, "Control Rod Block Instrumentation" for NUREG-1433 and NUREG-1434. The requirements for implementing the improved BPWS are described in General Electric Licensing Topical Report (LTR) NEDO-33091-A, Revision 2, "Improved BPWS Control Rod Insertion Process," dated July 2004. The General Electric Boiling Water Reactor Owner's Group (BWROG) participants in the Technical Specifications Task Force (TSTF) initially proposed these changes to the STS in TSTF-476, Revision 0, "Improved BPWS Control Rod Insertion Process (NEDO-33091).' TSTF-476, Revision 1 was submitted on January 9, 2007 and was later accepted by NRC. Hereafter, all references to TSTF-476 refer to TSTF-476, Revision 1, unless otherwise noted. Technical