

Commission: (1) If it is unable to comply with any of the requirements described in Attachment 1; (2) if compliance with any of the requirements is unnecessary in its specific circumstances; or (3) if implementation of any of the requirements would cause PG&E to be in violation of the provisions of any Commission regulation or the facility license. The notification shall provide PG&E's justification for seeking relief from or variation of any specific requirement.

2. If PG&E considers that implementation of any of the requirements described in Attachment 1 to this Order would adversely impact the safe storage of spent fuel, PG&E must notify the Commission, within twenty (20) days of this Order, of the adverse safety impact, the basis for its determination that the requirement has an adverse safety impact, and either a proposal for achieving the same objectives specified in the Attachment 1 requirements in question, or a schedule for modifying the facility to address the adverse safety condition. If neither approach is appropriate, PG&E must supplement its response to Condition B.1, of this Order, to identify the condition as a requirement with which it cannot comply, with attendant justifications as required under Condition B.1.

C.1. PG&E shall, within twenty (20) days of this Order, submit to the Commission a schedule for achieving compliance with each requirement described in Attachment 1.

2. PG&E shall report to the Commission when it has achieved full compliance with the requirements described in Attachment 1.

D. All measures implemented, or actions taken, in response to this Order, shall be maintained until the Commission determines otherwise.

PG&E's response to Conditions B.1, B.2, C.1, and C.2, above, shall be submitted in accordance with 10 CFR 72.4. In addition, submittals that contain Safeguards Information shall be properly marked and handled in accordance with 10 CFR 73.21.

The Director, Office of Nuclear Material Safety and Safeguards, may, in writing, relax or rescind any of the above conditions, for good cause.

IV

In accordance with 10 CFR 2.202, PG&E must, and any other entity adversely affected by this Order may, submit an answer to this Order, and may request a hearing on this Order, within twenty (20) days of the date of this Order. Where good cause is shown,

consideration will be given to extending the time to request a hearing. A request for extension of time in which to submit an answer must be made in writing to the Director, Office of Nuclear Material Safety and Safeguards, and the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and include a statement of good cause for the extension. The answer may consent to this Order. Unless the answer consents to this Order, the answer shall, in writing and under oath or affirmation, specifically set forth the matters of fact and law on which the licensee or other entity adversely affected relies and the reasons as to why the Order should not have been issued. Any answer or request for a hearing shall be submitted to the Secretary, Office of the Secretary of the Commission, U.S. Nuclear Regulatory Commission, ATTN: Rulemakings and Adjudications Staff, Washington, DC 20555. Copies also shall be sent to the Director, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555; to the Director, Office of Enforcement at the same address; to the Assistant General Counsel for Materials Litigation and Enforcement, at the same address; to the Regional Administrator for NRC Region IV at 611 Ryan Plaza Drive, Suite 400, Arlington, TX 76011; and to the licensee, if the answer or hearing request is by an entity other than the licensee. Because of possible disruptions in delivery of mail to United States Government offices, it is requested that requests for a hearing be transmitted to the Secretary of the Commission, either by means of facsimile transmission, to 301-415-1101, or by e-mail, to hearingdocket@nrc.gov, and also to the Office of General Counsel (OGC), either by means of facsimile transmission, to 301-415-3725, or by e-mail, to OGCMailCenter@nrc.gov. If an entity other than PG&E requests a hearing, that entity shall set forth, with particularity, the manner in which its interest is adversely affected by this Order, and shall address the criteria set forth in 10 CFR 2.309.

If PG&E or an entity whose interest is adversely affected requests a hearing, the Commission will issue an Order designating the hearing's time and place. If a hearing is held, the issue to be considered at such a hearing shall be whether this Order should be sustained.

Pursuant to 10 CFR 2.202(c)(2)(i), PG&E may, in addition to demanding a hearing at the time the answer is filed or sooner, move the presiding officer to set aside the immediate effectiveness of the Order on the grounds that the Order,

including the need for immediate effectiveness, is not based on adequate evidence but on mere suspicion, unfounded allegations, or error.

In the absence of any request for hearing or written approval of an extension of time in which to request a hearing, the provisions specified in Section III above shall be final twenty (20) days from the date of this Order, without further Order or proceedings. If an extension of time for requesting a hearing has been approved, the provisions specified in Section III shall be final when the extension expires, if a hearing request has not been received, an answer or a request for hearing shall not stay the immediate effectiveness of this order.

Dated at Rockville, Maryland this 22nd day of November 2006.

For the Nuclear Regulatory Commission.

Jack R. Strosnider,

Director, Office of Nuclear Material Safety.

[FR Doc. E6-20959 Filed 12-8-06; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 030-06172]

Notice of Availability of Environmental Assessment and Finding of No Significant Impact for License Amendment to Byproduct Materials License No. 37-07653-02, for Amendment of the License and Unrestricted Release of the Alcoa Inc.'s Facility in New Kensington, PA

AGENCY: Nuclear Regulatory Commission.

ACTION: Issuance of Environmental Assessment and Finding of No Significant Impact for License Amendment.

FOR FURTHER INFORMATION CONTACT:

Kathy Dolce Modes, Health Physicist, Materials Security & Industrial Branch, Division of Nuclear Materials Safety, Region I, 475 Allendale Road, King of Prussia, PA 19406-1415; (610)337-5251; fax number (610)337-5269; or by e-mail: kad@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is considering the issuance of a license amendment to Byproduct Materials License No. 37-07653-02. This license is held by Alcoa, Inc. (Formerly known as the Aluminum Company of America) (the Licensee), for its Alcoa Research Laboratory (the ARL Facility), located at Freeport Road in

New Kensington, Pennsylvania. Issuance of the amendment would authorize release of the ARL Facility for unrestricted use. The Licensee requested this action in a letter dated August 28, 2006. The NRC has prepared an Environmental Assessment (EA) in support of this proposed action in accordance with the requirements of Title 10, Code of Federal Regulations (CFR), Part 51 (10 CFR Part 51). Based on the EA, the NRC has concluded that a Finding of No Significant Impact (FONSI) is appropriate with respect to the proposed action. The amendment will be issued to the Licensee following the publication of this FONSI and EA in the **Federal Register**.

II. Environmental Assessment

Identification of Proposed Action

The proposed action would approve the Licensee's August 28, 2006, license amendment request, resulting in release of the ARL Facility for unrestricted use. License No. 37-07653-02 was issued on April 18, 1958, pursuant to 10 CFR Part 30, and has been amended periodically since that time. This license authorized the Licensee to use unsealed and sealed byproduct material for purposes of conducting research and development activities on laboratory bench tops and in hoods.

The Facility is situated on 14.126 acres in a residential area, and consists of office space and laboratories. Within the Facility, use of licensed materials was confined to 5,889 square feet in Building 29 and 2,320 square feet in Building 44.

On February 10, 2004, the Licensee ceased licensed activities and initiated a survey and decontamination of the ARL Facility. Based on the Licensee's historical knowledge of the site and the conditions of the Facility, the Licensee determined that only routine decontamination activities, in accordance with their NRC-approved, operating radiation safety procedures, were required. The Licensee was not required to submit a decommissioning plan to the NRC because worker cleanup activities and procedures are consistent with those approved for routine operations. The Licensee conducted surveys of the Facility and provided information to the NRC to demonstrate that it meets the criteria in Subpart E of 10 CFR Part 20 for unrestricted release.

Need for the Proposed Action

The Licensee has ceased conducting licensed activities at the Facility, and seeks the unrestricted use of its ARL Facility.

Environmental Impacts of the Proposed Action

The historical review of licensed activities conducted at the Facility shows that such activities involved use of the following radionuclides with half-lives greater than 120 days: Hydrogen-3, sodium-22, aluminum-26, calcium-45, manganese-54, iron-55, cobalt-60, nickel-63, zinc-65, strontium-90, cadmium-109, antimony-125, cesium-137, and thallium-204. Prior to performing the final status survey, the Licensee conducted decontamination activities, as necessary, in the areas of the Facility affected by these radionuclides.

The Licensee conducted final status surveys in Buildings 29 and 44 in 1971, 2004, and 2006 and attached a final status survey report to their amendment request dated August 28, 2006. The Licensee elected to demonstrate compliance with the radiological criteria for unrestricted release as specified in 10 CFR 20.1402 by using the screening approach described in NUREG-1757, "Consolidated NMSS Decommissioning Guidance," Volume 2. The Licensee used the radionuclide-specific derived concentration guideline levels (DCGLs), developed there by the NRC, which comply with the dose criterion in 10 CFR 20.1402. These DCGLs define the maximum amount of residual radioactivity on building surfaces, equipment, and materials, and in soils, that will satisfy the NRC requirements in Subpart E of 10 CFR Part 20 for unrestricted release. The Licensee's final status survey results were below these DCGLs and are in compliance with the As Low As Reasonably Achievable (ALARA) requirement of 10 CFR 20.1402. The NRC thus finds that the Licensee's final status survey results are acceptable.

Based on its review, the staff has determined that the affected environment and any environmental impacts associated with the proposed action are bounded by the impacts evaluated by the "Generic Environmental Impact Statement in Support of Rulemaking on Radiological Criteria for License Termination of NRC-Licensed Nuclear Facilities" (NUREG-1496) Volumes 1-3 (ML042310492, ML042320379, and ML042330385). The staff finds there were no significant environmental impacts from the use of radioactive material at the Facility. The NRC staff reviewed the docket file records and the final status survey report to identify any non-radiological hazards that may have impacted the environment surrounding the Facility. No such hazards or impacts to the

environment were identified. The NRC has identified no other radiological or non-radiological activities in the area that could result in cumulative environmental impacts.

The NRC staff finds that the proposed release of the Facility for unrestricted use and the amendment of the NRC materials license is in compliance with 10 CFR 20.1402. Based on its review, the staff considered the impact of the residual radioactivity at the Facility and concluded that the proposed action will not have a significant effect on the quality of the human environment.

Environmental Impacts of the Alternatives to the Proposed Action

Due to the largely administrative nature of the proposed action, its environmental impacts are small. Therefore, the only alternative the staff considered is the no-action alternative, under which the staff would leave things as they are by simply denying the amendment request. This no-action alternative is not feasible because it conflicts with 10 CFR 30.36(d), requiring that decommissioning of byproduct material facilities be completed and approved by the NRC after licensed activities cease. The NRC's analysis of the Licensee's final status survey data confirmed that the ARL Facility meets the requirements of 10 CFR 20.1402 for unrestricted release. Additionally, denying the amendment request would result in no change in current environmental impacts. The environmental impacts of the proposed action and the no-action alternative are therefore similar, and the no-action alternative is accordingly not further considered.

Conclusion

The NRC staff has concluded that the proposed action is consistent with the NRC's unrestricted release criteria specified in 10 CFR 20.1402. Because the proposed action will not significantly impact the quality of the human environment, the NRC staff concludes that the proposed action is the preferred alternative.

Agencies and Persons Consulted

NRC provided a draft of this Environmental Assessment to the Commonwealth of Pennsylvania for review on October 12, 2006. On October 27, 2006, Commonwealth of Pennsylvania responded by e-mail (ML063000472). The State agreed with the conclusions of the EA, and otherwise had no comments.

The NRC staff has determined that the proposed action is of a procedural nature, and will not affect listed species

or critical habitat. Therefore, no further consultation is required under Section 7 of the Endangered Species Act. The NRC staff has also determined that the proposed action is not the type of activity that has the potential to cause effects on historic properties. Therefore, no further consultation is required under Section 106 of the National Historic Preservation Act.

III. Finding of No Significant Impact

The NRC staff has prepared this EA in support of the proposed action. On the basis of this EA, the NRC finds that there are no significant environmental impacts from the proposed action, and that preparation of an environmental impact statement is not warranted. Accordingly, the NRC has determined that a Finding of No Significant Impact is appropriate.

IV. Further Information

Documents related to this action, including the application for license amendment and supporting documentation, are available electronically at the NRC's Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html>. From this site, you can access the NRC's Agencywide Document Access and Management System (ADAMS), which provides text and image files of NRC's public documents. The documents related to this action are listed below, along with their ADAMS accession numbers.

NUREG-1757, "Consolidated NMSS Decommissioning Guidance;"

Title 10 Code of Federal Regulations, Part 20, Subpart E, "Radiological Criteria for License Termination;"

Title 10, Code of Federal Regulations, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions;"

NUREG-1496, "Generic Environmental Impact Statement in Support of Rulemaking on Radiological Criteria for License Termination of NRC-Licensed Nuclear Facilities;"

Licensee letter dated June 21, 2005 and attachments—first request to remove ARL facility from license (ML051920272); and

Licensee letter dated August 28, 2006 and attachments—final request to remove ARL facility from license (ML062550071).

If you do not have access to ADAMS, or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov. These documents may also be viewed electronically on the public computers located at the NRC's PDR, O 1 F21, One

White Flint North, 11555 Rockville Pike, Rockville, MD 20852. The PDR reproduction contractor will copy documents for a fee.

Dated at U.S. NRC Region I Office located in King of Prussia, Pennsylvania this 4th day of December 2006.

For the Nuclear Regulatory Commission,
Marie Miller,
Chief, Materials Security and Industrial Branch, Division of Nuclear Materials Safety, Region I.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 030-12998]

Notice of Availability of Environmental Assessment and Finding of No Significant Impact for License Amendment to Byproduct Materials License No. 37-07438-15, for the Unrestricted Release of the Philadelphia Health & Education Corporation's Facility in Doylestown, PA

AGENCY: Nuclear Regulatory Commission.

ACTION: Issuance of Environmental Assessment and Finding of No Significant Impact for License Amendment.

FOR FURTHER INFORMATION CONTACT:

Dennis Lawyer, Health Physicist, Commercial and R&D Branch, Division of Nuclear Materials Safety, Region 1, 475 Allendale Road, King of Prussia, Pennsylvania; telephone (610) 337-5366; fax number (610) 337-5393; or by e-mail: drl1@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is considering the issuance of a license amendment to Byproduct Materials License No. 37-07438-15. This license is held by Philadelphia Health & Education Corporation, d/b/a/ Drexel University College of Medicine (the Licensee), for the area leased to the Licensee within the Delaware Valley College of Agriculture and Science's Mandrell Science Building (the Facility), located at 700 E. Butler Avenue in Doylestown, Pennsylvania. Issuance of the amendment would authorize release of the Facility for unrestricted use. The Licensee requested this action in a letter dated August 28, 2006. The NRC has prepared an Environmental Assessment (EA) in support of this proposed action

in accordance with the requirements of Title 10, Code of Federal Regulations (CFR), Part 51 (10 CFR Part 51). Based on the EA, the NRC has concluded that a Finding of No Significant Impact (FONSI) is appropriate with respect to the proposed action. The amendment will be issued to the Licensee following the publication of this FONSI and EA in the **Federal Register**.

II. Environmental Assessment

Identification of Proposed Action

The proposed action would approve the Licensee's August 28, 2006, license amendment request, resulting in release of the Facility for unrestricted use. License No. 37-07438-15 was issued on July 17, 1977, pursuant to 10 CFR Part 30 and has been amended periodically since that time. This license authorized the Licensee to use unsealed byproduct material for purposes of conducting research and development activities on laboratory bench tops and in hoods.

The Facility is a 15,000 square foot leased area, within the 66,300 square foot Mandrell Science Building, located on the 80 acre Delaware Valley College of Agriculture and Science Campus. The Facility consists of office space and laboratories. Within the Facility, use of licensed materials was confined to laboratories totaling 2,680 square feet.

On July 26, 2006, the Licensee ceased licensed activities and initiated a survey and decontamination of the Facility. Based on the Licensee's historical knowledge of the site and the conditions of the Facility, the Licensee determined that only routine decontamination activities, in accordance with their NRC-approved, operating radiation safety procedures, were required. The Licensee was not required to submit a decommissioning plan to the NRC because worker cleanup activities and procedures are consistent with those approved for routine operations. The Licensee conducted surveys of the Facility and provided information to the NRC to demonstrate that it meets the criteria in Subpart E of 10 CFR Part 20 for unrestricted release.

Need for the Proposed Action

The Licensee has ceased conducting licensed activities at the Facility, and seeks release of the Facility for unrestricted use.

Environmental Impacts of the Proposed Action

The historical review of licensed activities conducted at the Facility shows that such activities involved use of hydrogen-3, which has a half-life greater than 120 days. Prior to